

# **Fast Neutron Direct Interrogation (FNDI)**

**International Partnership for Nuclear Disarmament  
Verification (IPNDV) Working Group 3**

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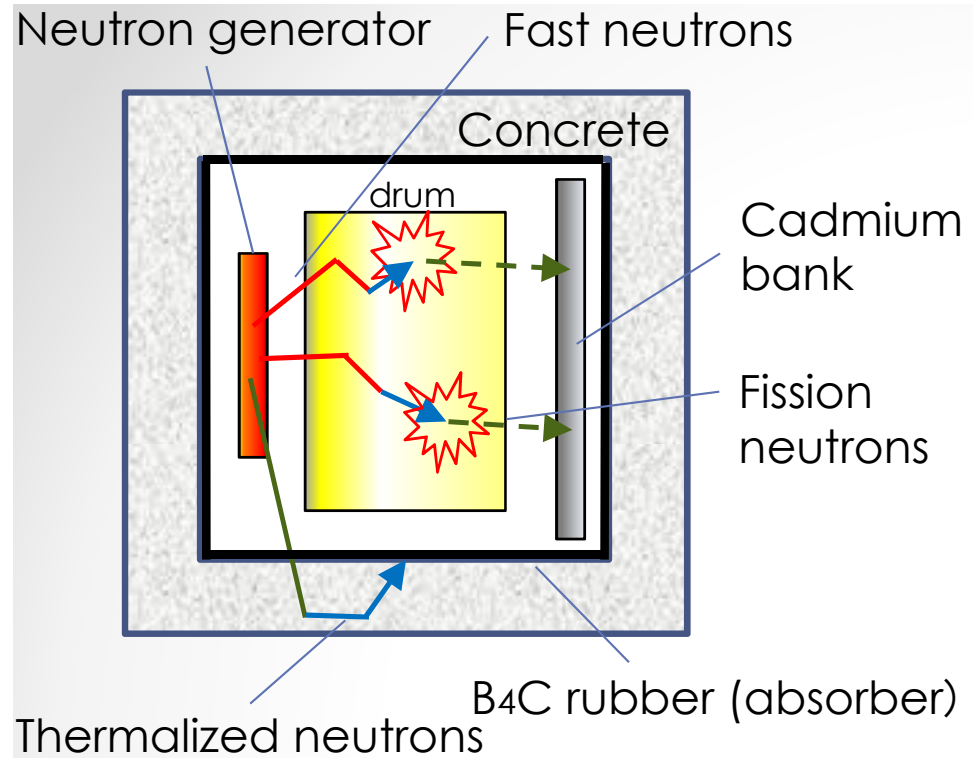
**Japan Atomic Energy Agency**

# FNDI method (Outline)

Fast Neutron Direct Interrogation (FNDI) method:

⇒ **Active neutron NDA method for quantifying fissile materials**

- Source:  
Pulsed Fast Neutron (14MeV)
- Detector:  
 $^3\text{He}$  Detector (Cadmium Bank)
- Less positional dependence of fissile materials (U, Pu) in monitoring targets (drums)

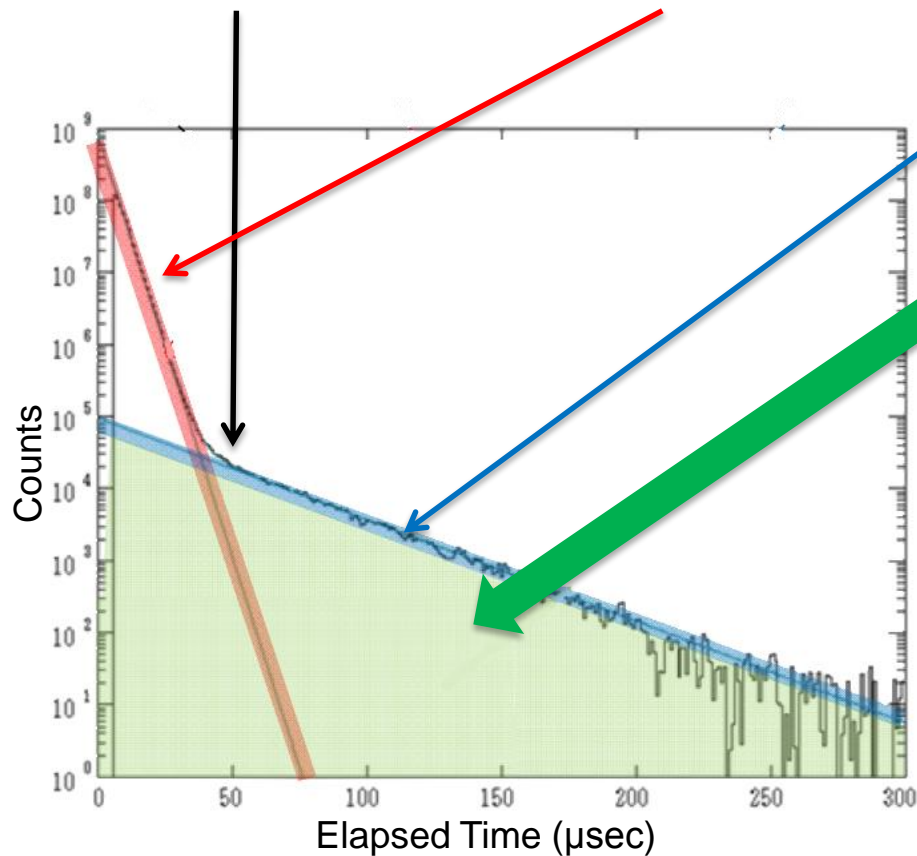


(Outline of System Configuration)

# FNDI method (Physical Principles)

$$S(t) = a_0 + a_1 \times \exp(-\lambda_1 t) + a_2 \times \exp(-\lambda_2 t) + \dots + a_n \times \exp(-\lambda_n t)$$

Measurement    Interrogation Neutrons    Fission Neutrons



Integrated Fission Neutrons

$\propto$  Fissile Materials Mass

- Measurement data is separated into the two components by non-linear least square method.
- Fission neutrons component can be simply separated.

# FNDI method (Example: JAWAS-N)

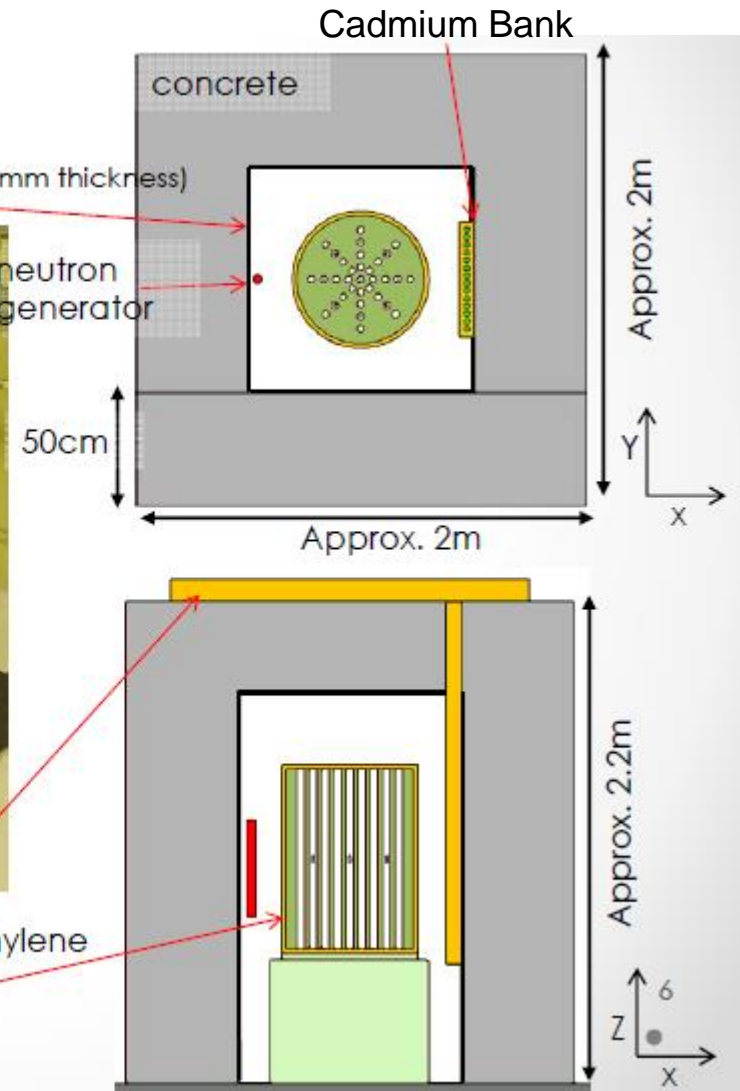
**JAWAS-N:**  
**J**AEA **A**ctive **W**aste **A**ssay **S**ystem - **N**ingyo



Basic characteristic tests have been tried since 2014.

Drum with PMD※

※Polyethylene Moderator



# FNDI method (Example: JAWAS-N)



## Inner Structure (1Hx1Dx1.25H [m])

- (Left) D-T Neutron Generator:  
 $10^8$  n/sec, 14MeV
- (Middle) Drum Rotator: 2.5 rpm
- (Right) Cadmium Bank
  - Surface: 2mm Cd-plate
  - 14  $^3\text{He}$ -detectors
  - Polyethylene moderator is filled between the Cd-plate and detectors.



## Measurement Control Unit

- HV supply
- Timing Pulse Generator
- MCS (Multichannel Scaler)
- PC (control/data log)
  - Analysis System: JAEA original

# FNDI method (Performance/Applicability)

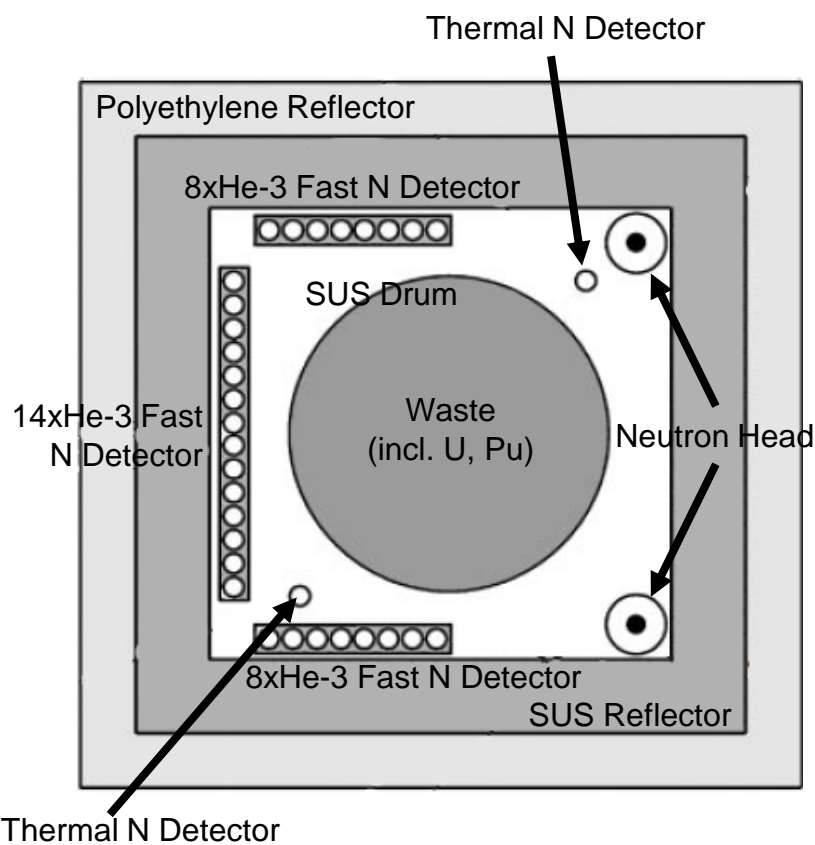
- Performance/Applicability Example (JAWAS-N experiences)

| Items                                          | Chemical Form                                                                                                     | Applicability |
|------------------------------------------------|-------------------------------------------------------------------------------------------------------------------|---------------|
| Scrap uranium                                  | UF <sub>4</sub>                                                                                                   | OK            |
| Uranium absorbed with aluminum oxides pellets  | UF <sub>4</sub> , UO <sub>2</sub> F <sub>2</sub>                                                                  | OK            |
| Uranium absorbed with sodium fluorides pellets | UF <sub>4</sub>                                                                                                   | OK            |
| Uranium oxides powders                         | UO <sub>2</sub> , UO <sub>3</sub><br>Mixture of UO <sub>2</sub> , UO <sub>3</sub> , U <sub>3</sub> O <sub>8</sub> | OK            |
| Uranium absorbed with calcium precipitates     | CaF <sub>2</sub> -CaU <sub>2</sub> O <sub>7</sub>                                                                 | NG            |
| Small amount of sample                         | UF <sub>6</sub> , UF <sub>4</sub>                                                                                 | OK            |
| Others (included solutions)                    | UO <sub>2</sub> (SO <sub>4</sub> ) <sub>2</sub> , UO <sub>2</sub> (NO <sub>3</sub> ) <sub>2</sub>                 | NG            |

| Demand             | Features                                                             |
|--------------------|----------------------------------------------------------------------|
| Objective Packages | 200 litter drum (metal containments)                                 |
| Detective Range    | 3gU-200kgU                                                           |
| Accuracy           | <20% (in case of <1kgU), <1% (in case of >1kgU)                      |
| Measuring Time     | 10min with additional time (10-15min in case of gamma-ray detection) |

# FNDI method (Performance/Applicability 2)

- Expected Detection Limit for Pu (Evaluated by simulation code)



Simulation Model

| Waste Type                | Target Material | Detection Limit |                   |        |
|---------------------------|-----------------|-----------------|-------------------|--------|
|                           |                 | mg              | Bq/g ( $\alpha$ ) | PPB    |
| Cementation* <sup>1</sup> | 60% Pu          | 0.0186          | 0.545             | 0.0387 |
|                           | 95% Pu          | 0.0134          | 0.0739            | 0.0279 |
|                           | Natural U       | 2.31            | 0.000122          | 4.82   |
| Metallic* <sup>2</sup>    | 60% Pu          | 0.0478          | 3.12              | 0.222  |
|                           | 95% Pu          | 0.0345          | 0.424             | 0.161  |
|                           | Natural U       | 5.96            | 0.000702          | 27.7   |
| Cellulose* <sup>3</sup>   | 60% Pu          | 0.0121          | 2.00              | 0.142  |
|                           | 95% Pu          | 0.0875          | 0.272             | 0.103  |
|                           | Natural U       | 1.51            | 0.000451          | 17.7   |

\*1 Total weight: 480kg, 10min of measurement.

\*2 Total weight: 215.6kg, 10min of measurement.

\*3 Total weight: 85kg, 10min of measurement.

?: <sup>239</sup>Pu density