



Pulsed Neutron Interrogation Test Assembly - PUNITA

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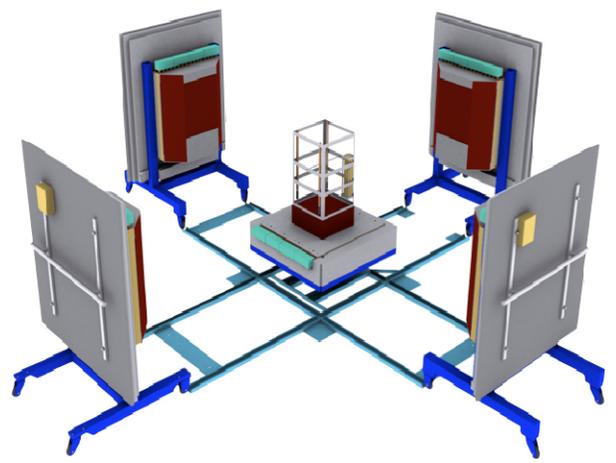
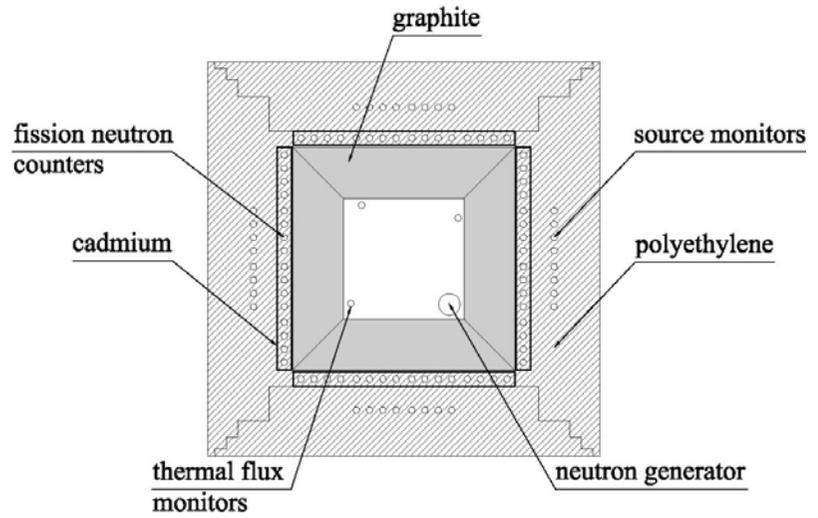
presented at

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JRC, Ispra site

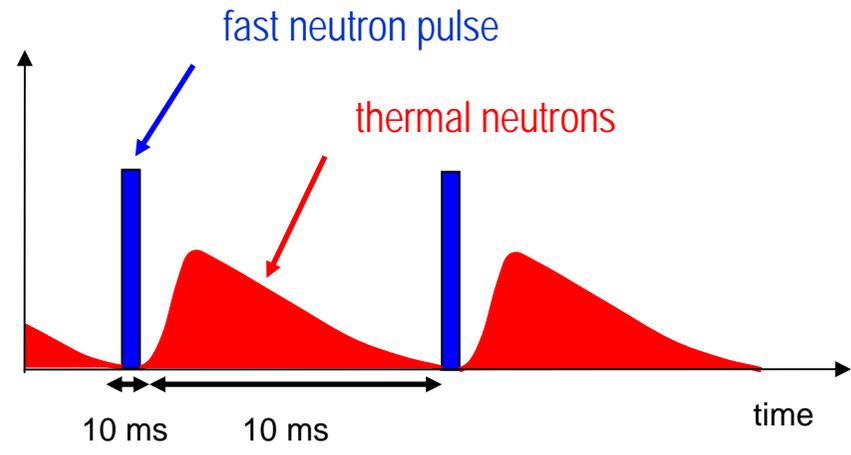
Outline

- characteristics of the Pulsed Neutron Interrogation Test Assembly
- some examples of research applications at PUNITA
 - fissile mass - differential die-away technique
 - design of a compact pulsed neutron device
 - explosives detection
 - detection of SNM
- useful signatures for nuclear disarmament verification

Pulsed Neutron Interrogation Test Assembly - PUNITA



- 205 mm thick graphite linear
- 96 fission neutron counters: He-3 in HDPE
- 32 monitors for neutron generator output

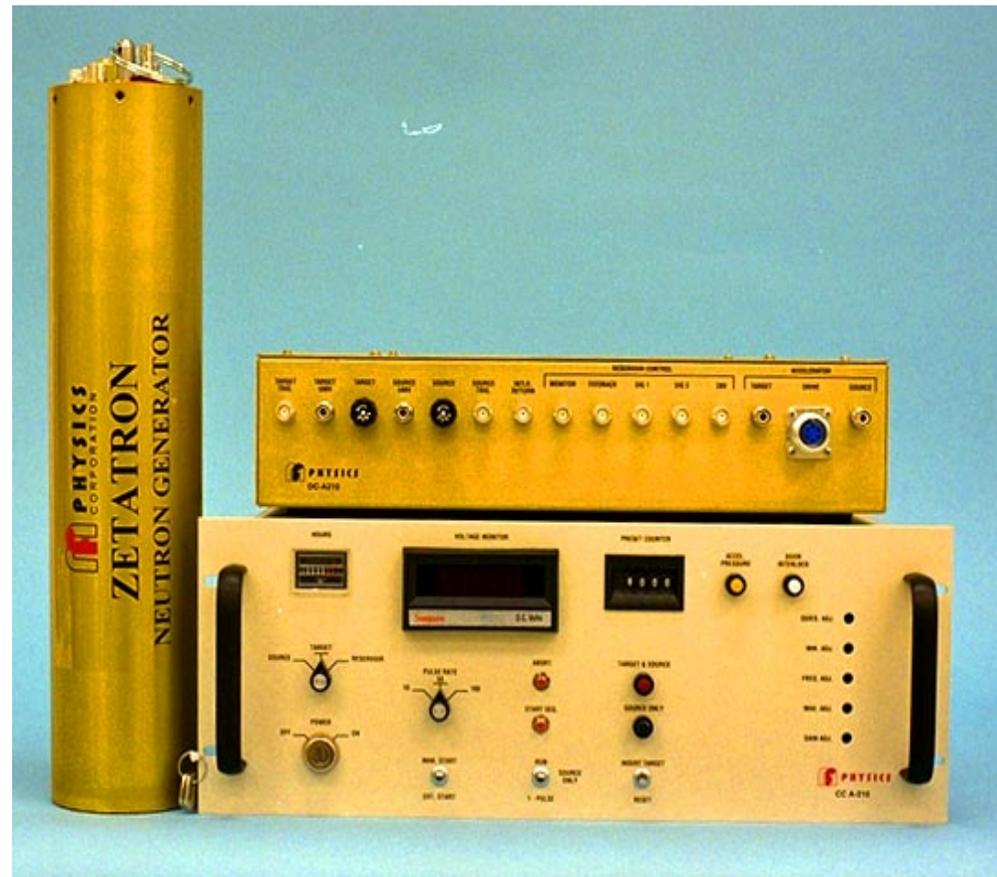


- 14-MeV neutron generator (MF Physics Model A-211)
- sealed, D-T mixed beam
- pulsing: 100 s⁻¹



MF Physics Model A-211, 14-MeV neutron generator

- neutron emission:
 $2 \cdot 10^8$ /s, 10^6 /burst in 4π angle
- sealed, D-T mixed beam
300-500 hours life
- pulsing of ion source and
acceleration voltage
no emission between bursts
- neutron burst width:
 $5-10 \mu\text{s}$
- burst repetition rate:
 $10 - 150 \text{ s}^{-1}$

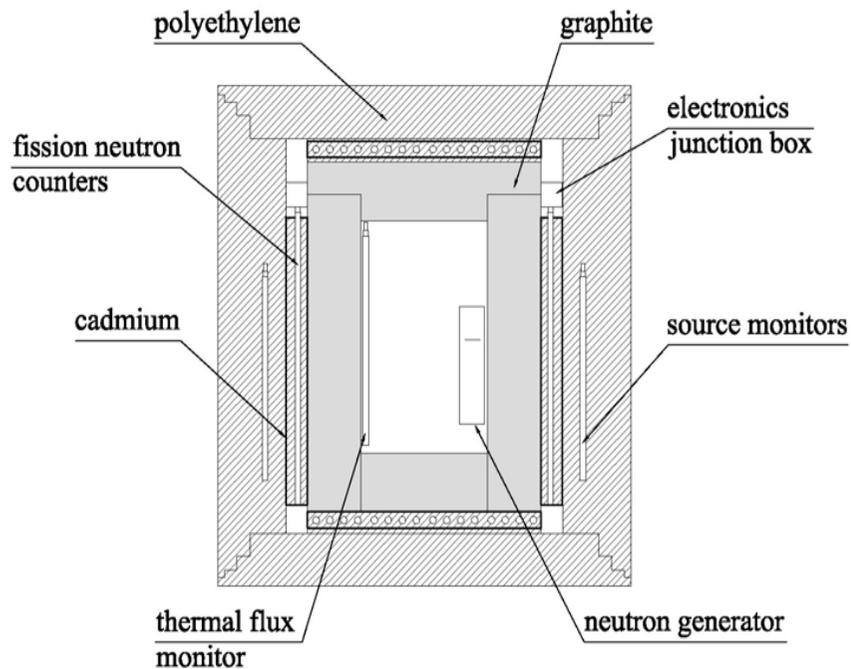


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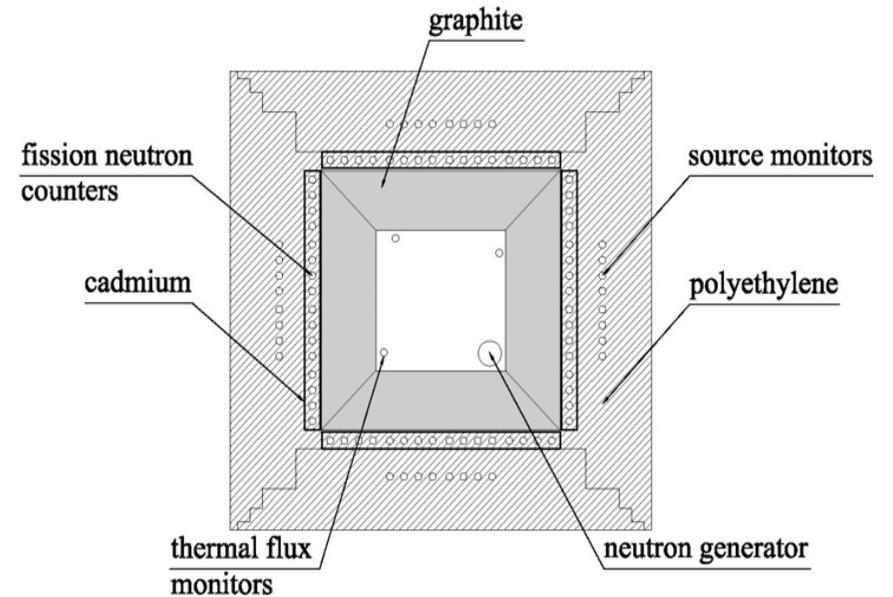


Features

- sample cavity: 50 cm x 50 cm x 80 cm
- thick graphite linear on all six side
- fission neutron counters: He-3 in polyethylene
- low pressure, thermal flux monitors
- monitor for n-generator output



side view



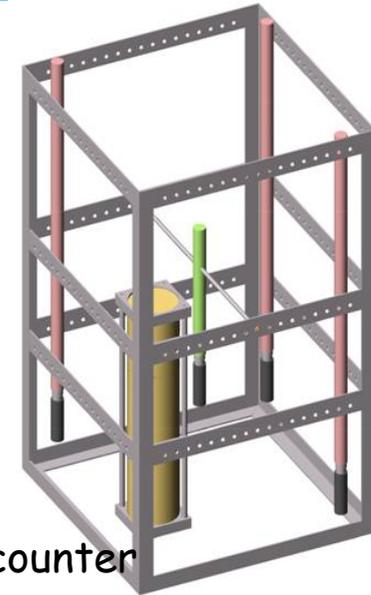
top view

Pulsed Neutron Interrogation Test Assembly - PUNITA

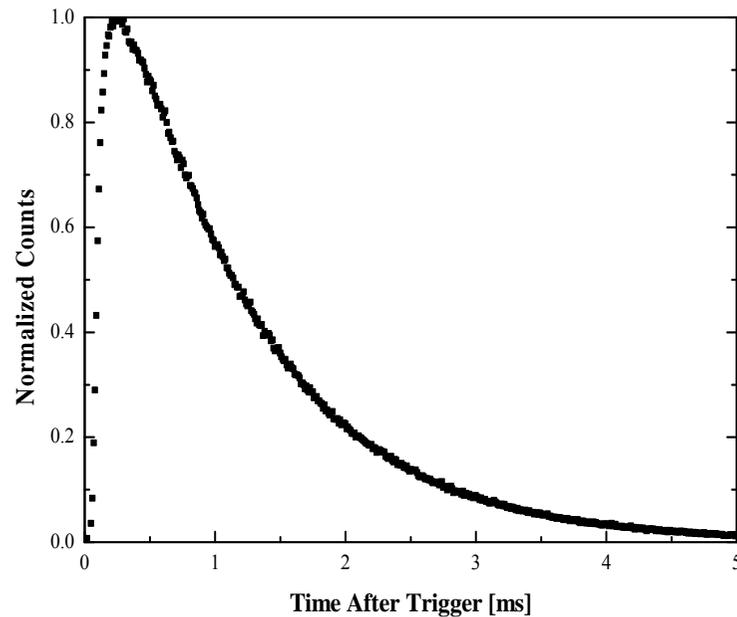


Exponential time response function of thermal neutrons

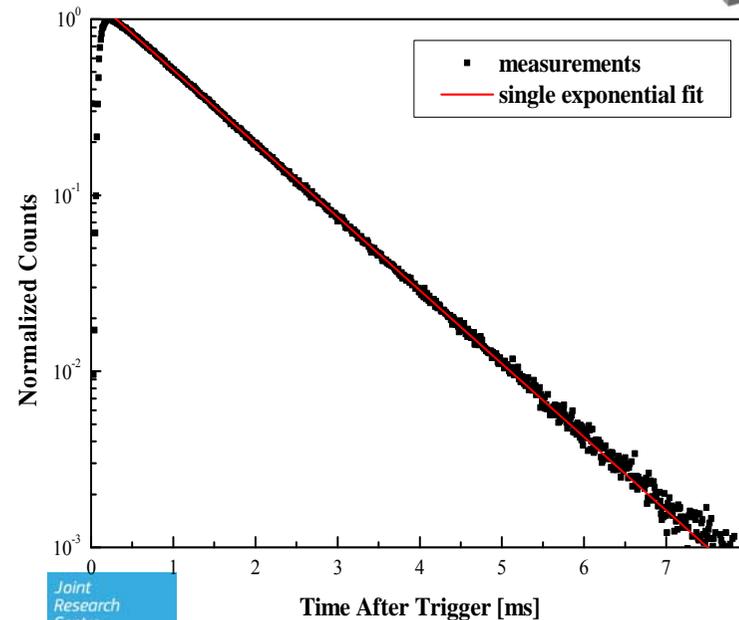
- maximum thermal flux at $280 \mu\text{s}$ after 14-MeV burst
- thermal neutron lifetime of 1.05 ± 0.02 milliseconds



^{235}U fission chamber



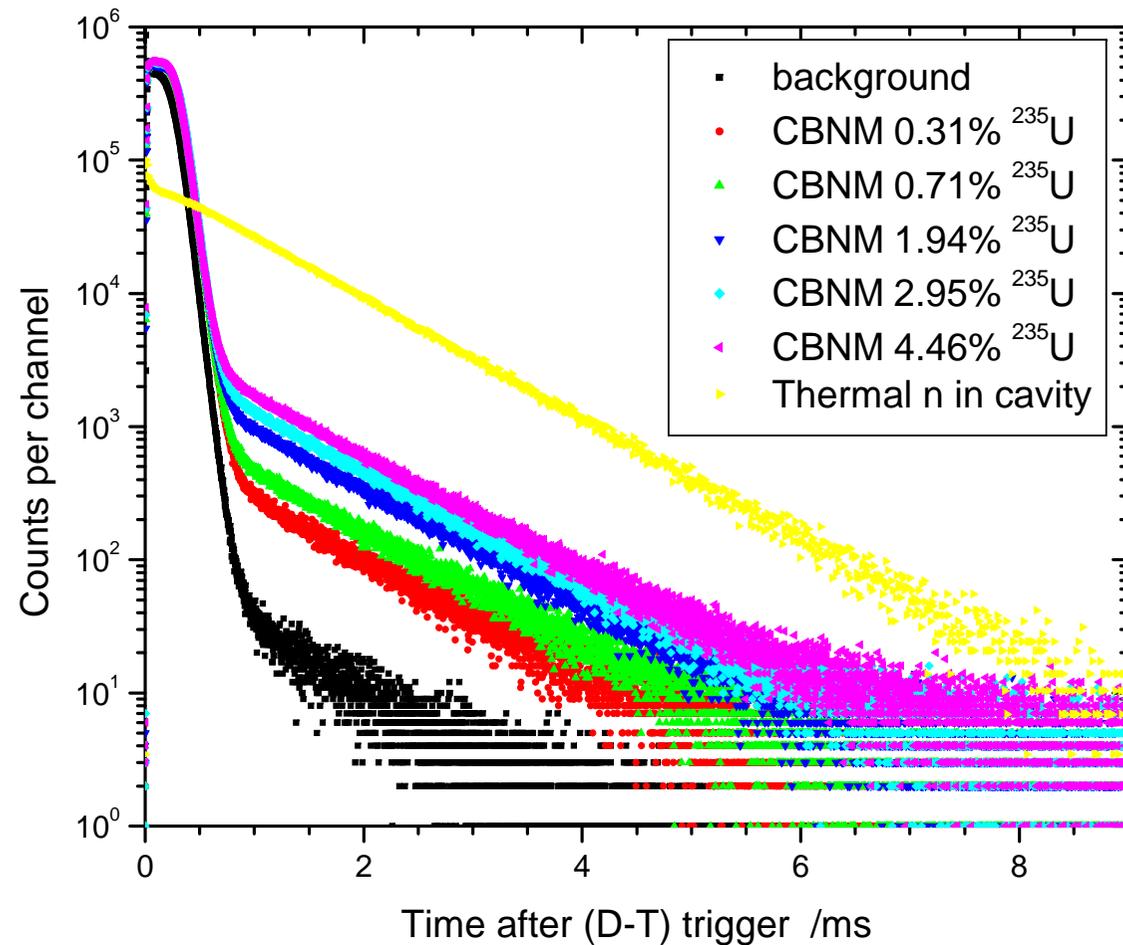
Small He-3 proportional counter



Detection of prompt fission neutrons - Differential Die-Away technique (DDA)

CBNM U_3O_8 standards

^{235}U mass range: 0.5 – 7.5 g
Meas. time: 10 min
14-MeV rate: 10^7 /s
Pulse rate: 100 Hz
Detectors: 3-He



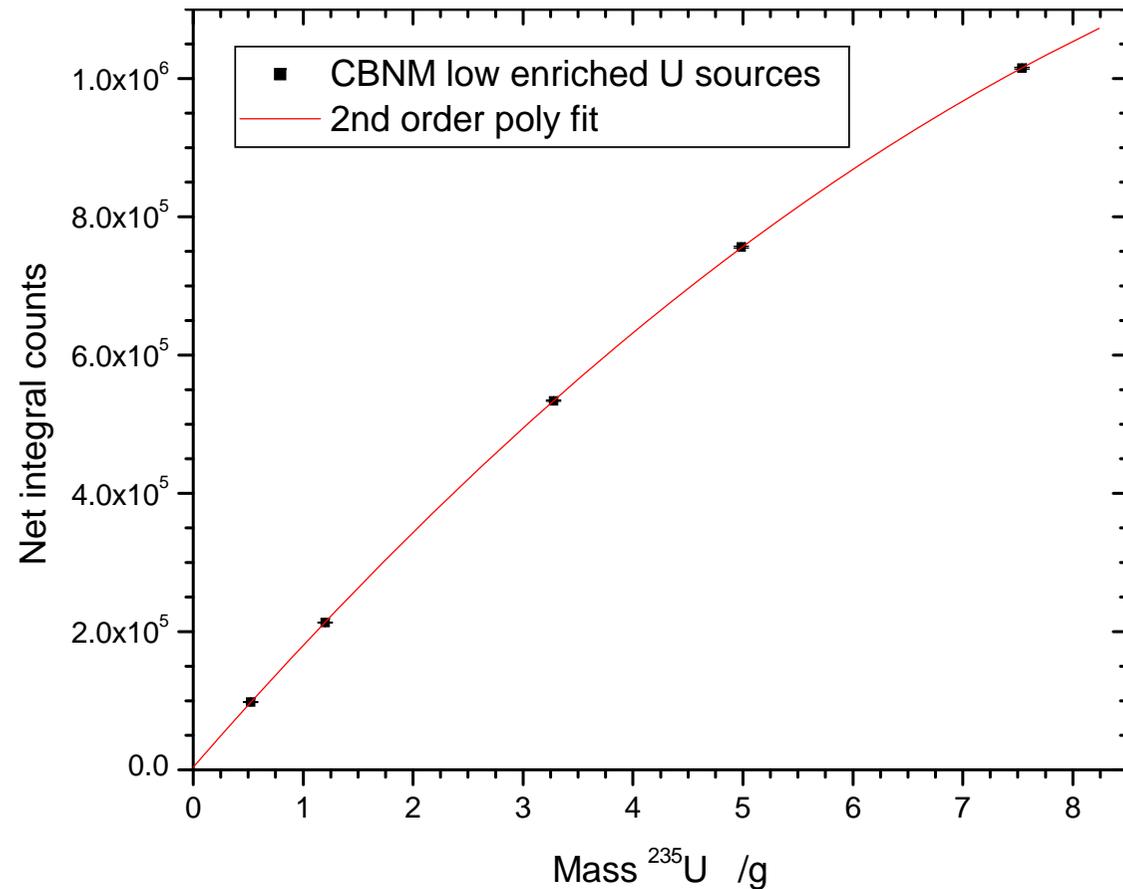


Detection of prompt fission neutrons - Differential Die-Away technique (DDA)

CBNM U_3O_8 standards

Integral range: 700 – 4700 μs

^{235}U mass range: 0.5 – 7.5 g
Meas. time: 10 min
14-MeV rate: 10^7 /s
Pulse rate: 100 Hz
Detectors: 96 3He



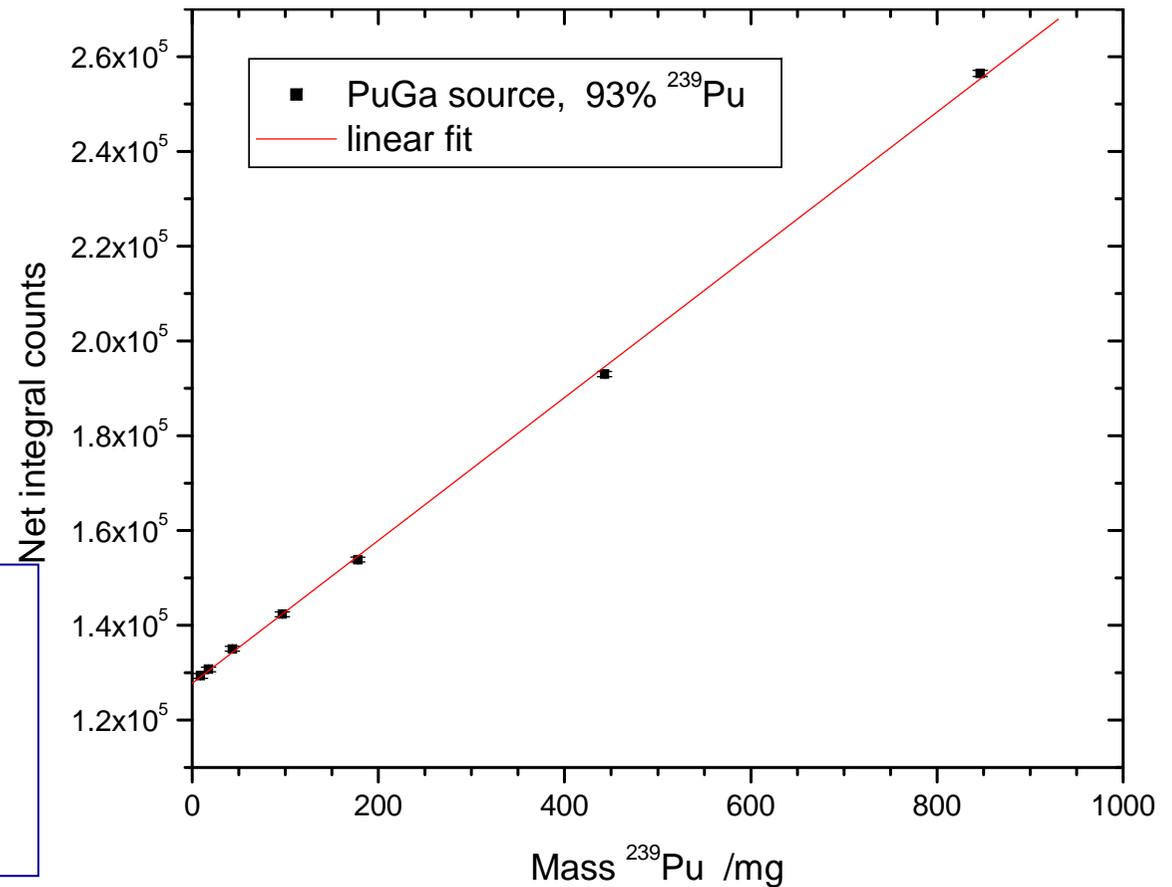


Detection of prompt fission neutrons - Differential Die-Away technique (DDA)

PuGa standards

Integral range: 700 – 4700 μ s

^{239}Pu mass range: 8.6 – 846 mg
Meas. time: 10 min
14-MeV rate: 10^7 /s
Pulse rate: 100 Hz
Detectors: 3-He





Can we design a compact device for mass assay of small fissile samples by active neutron interrogation?

Purpose:

mass assay of U and Pu samples of small volume

- e.g. smear samples, process samples, liquid solutions
- for accountancy, transport declarations, inventory

Advantages in “compact” design:

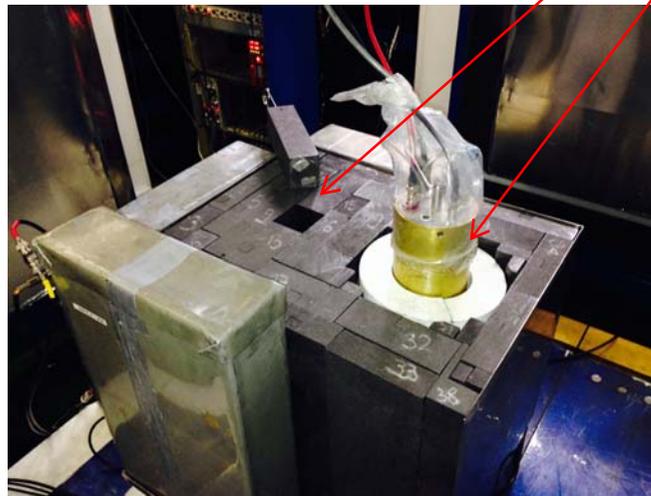
- less shielding
- less ^3He detector volume
- integrate with existing equipment
- lower price

Requirements:

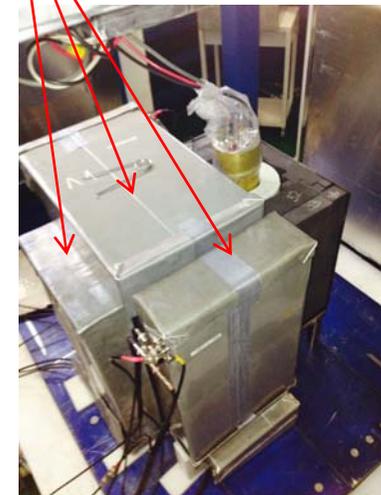
- high neutron flux at sample position
- long life time in moderator, short life time in detector
- high detection efficiency for fission neutrons

Experimental setup

- 5 detector banks, each composed of:
 - 5 ^3He detectors, polyethylene moderator, complete Cd cover
 - fission neutron detection efficiency: 11.6%
- graphite moderator block:
 - reactor grade purity, density 1.67 g/cm^3
 - dimensions: length: 509mm, height: 420mm, width: 402mm
 - distance generator target – sample: 240 mm



sample cavity
neutron generator
detector banks





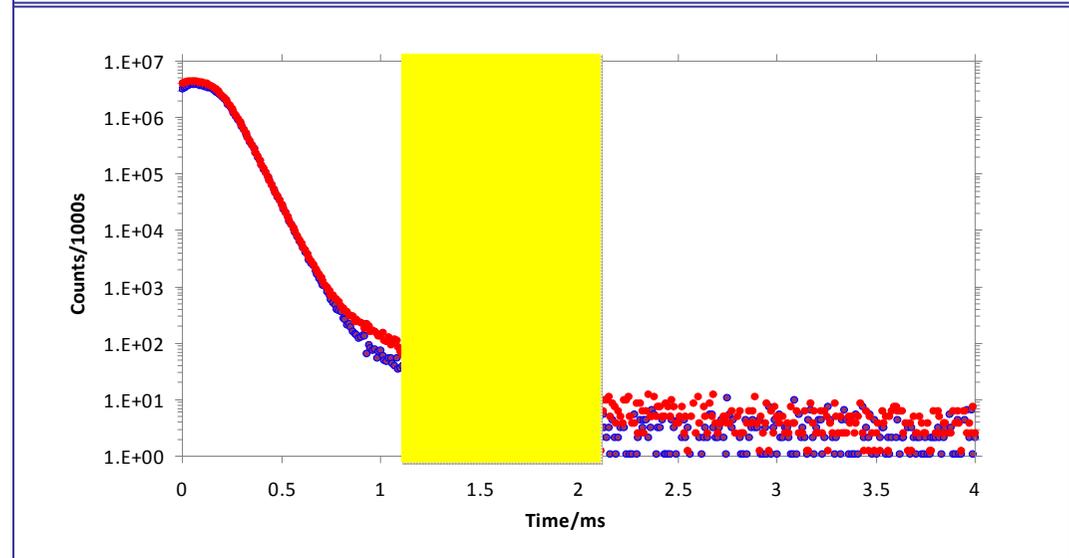
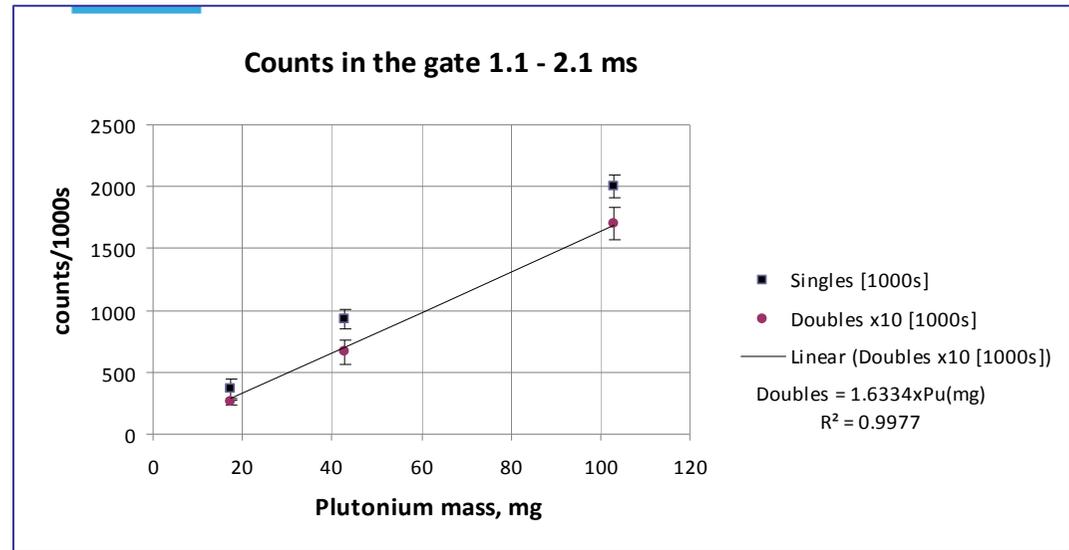
Experimental results

^{239}Pu Limit of Detection,
3 sigma of background,
meas time 1000s:

- Singles: $\text{LOD}_S = 0.786 \text{ mg}$
- Doubles: $\text{LOD}_D = 5.170 \text{ mg}$

using:
$$\text{LOD} = \frac{3\sqrt{\text{background}}}{\text{regression slope}}$$

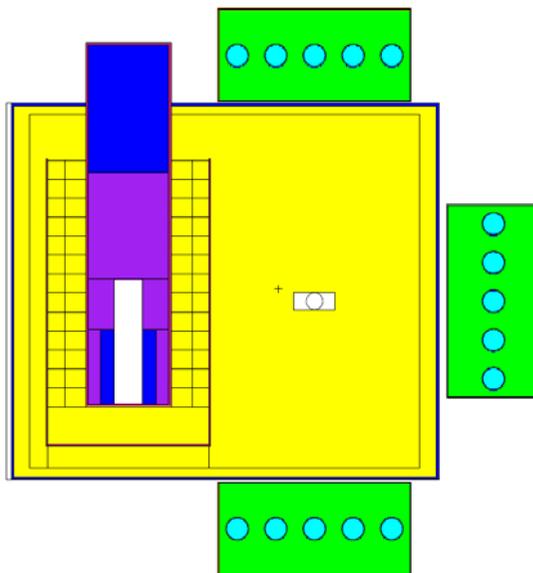
@ neutron emission rate:
 $0.9 \cdot 10^7 \text{ s}^{-1}$,
(max rate $2.0 \cdot 10^8 \text{ s}^{-1}$)



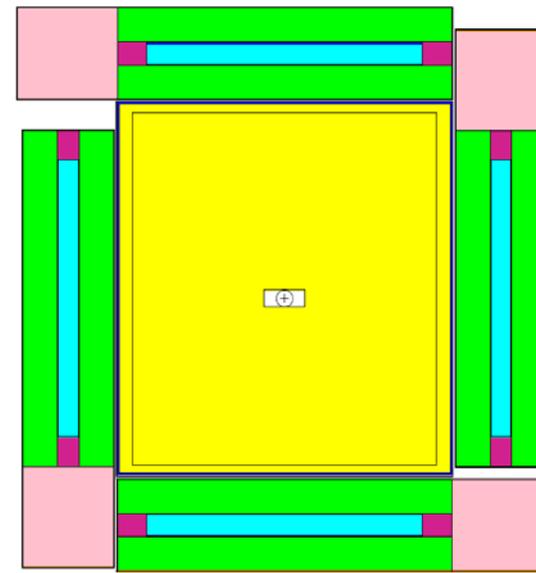


MCNP simulations, experimental configuration

- 5 detector banks (standard “neutron collar”), each composed of:
5 ^3He detectors, polyethylene moderator, complete Cd cover
fission neutron detection efficiency: 11.6%
- graphite moderator block: reactor grade purity, density 1.67 g/cm^3
dimensions: length: 509mm, height: 420mm, width: 402mm
distance target–sample: 240 mm



vertical cross section



vertical cross section



MCNP simulations, improved geometry

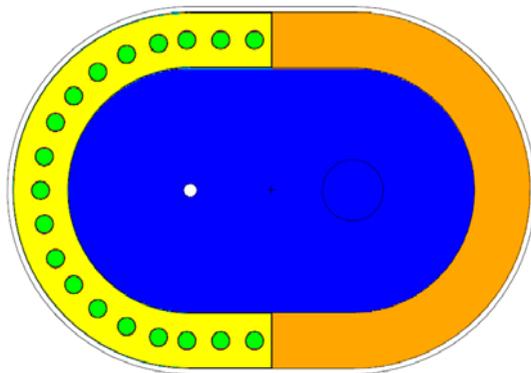
- better detector position, ^3He in HDPE, Cd cover, 80 mm width
- only 20% more ^3He volume

Moderator zone: max length 600 mm, max width 360 mm, height 360 mm

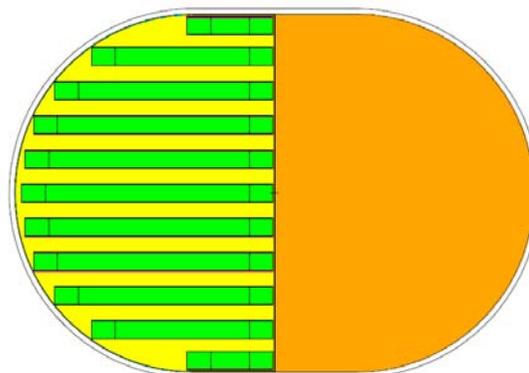
Reflector zone: width 80 mm

14-MeV source: 45 mm radius for W filter

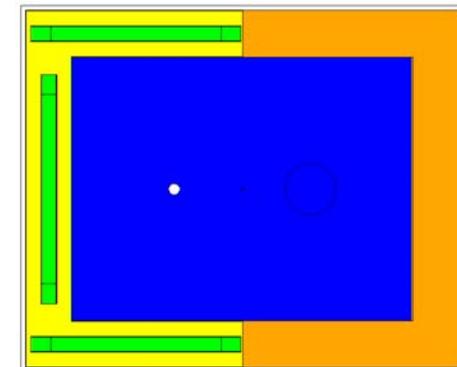
Sample cavity: void, radius 10 mm, distance to source 240 mm



horizontal cross section



horizontal cross section

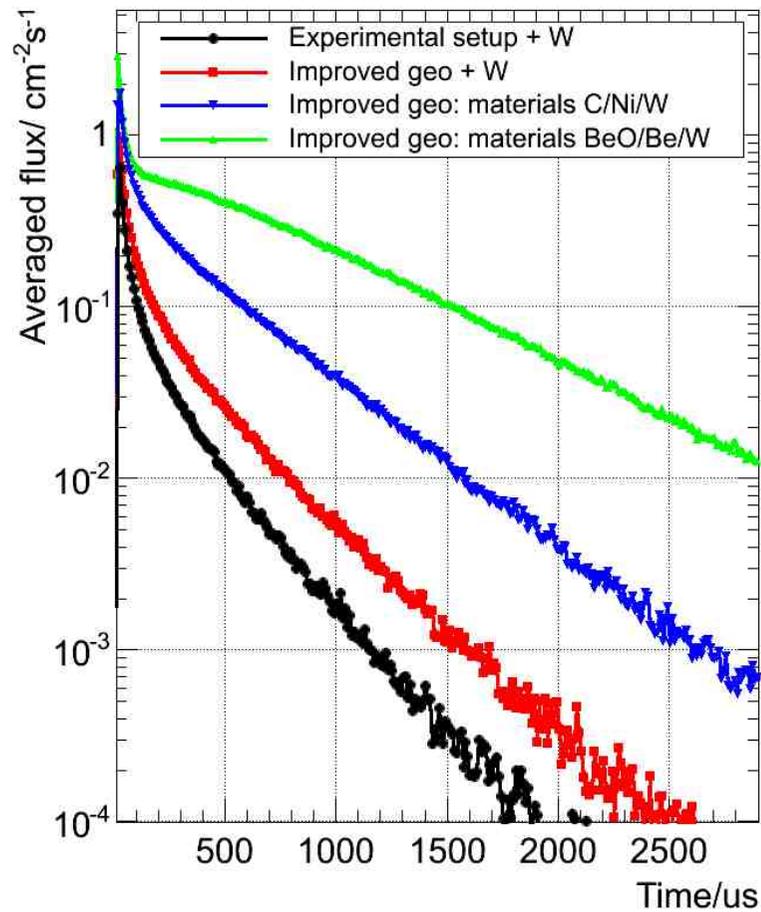


vertical cross section

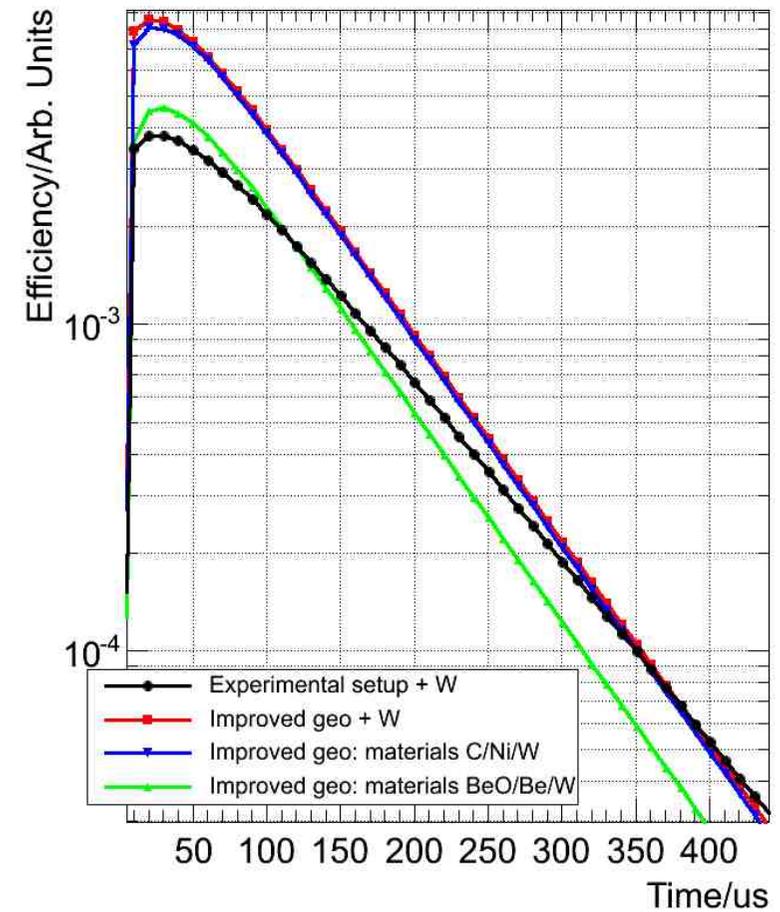


MCNP simulations

Flux at sample position



Detection of fission neutrons





MCNP simulations,

Integral configurations

Model	$\epsilon/\epsilon_{\text{ref}}$	Φ/Φ_{ref}	FOM
Experimental configuration	0.529	1.14	0.603
Experimental configuration, W around 14-MeV source	0.526	1.77	0.931
Improved geometry	1	1	1
Improved geometry with W around 14-MeV source	1.00	3.20	3.20
Improved geometry, BeO moderator, Be reflector, W	0.555	17.48	9.70
Improved geometry, C $\rho = 2.07 \text{ g/cm}^3$, Ni reflector, W	0.960	9.22	8.85



What “limit of detection” can we achieve in the compact design?

$$LOD_{improved} = LOD_{experimental} \frac{FOM_{experimental}}{FOM_{improved}} = 785 \mu g \frac{0.603}{8.85} = 53.6 \mu g$$

Assuming:

conservative design:

- graphite moderator of $\rho = 2.07 \text{ g/cm}^3$
- Ni reflector
- W on 14-MeV source

conservative generator operation: neutron emission rate: $0.9 \cdot 10^7 \text{ s}^{-1}$
(max rate $2.0 \cdot 10^8 \text{ s}^{-1}$)



Prompt Gamma Neutron Activation Analysis - PGNA

Method

- Interrogation first by fast neutrons later by thermal neutrons
- Detection of characteristic gamma rays from inelastic scattering and capture
- Known detector efficiency, nuclear data, and net peak areas yields element ratios
- Interesting elements for explosives detection are:
nitrogen, carbon, hydrogen, oxygen, chlorine, fluorine, a.o.

Equipment

- Detector shielding against thermal neutrons and gamma background
- Lanthanum bromide scintillation detectors ($1\frac{1}{2}'' \times 1\frac{1}{2}''$ and $2'' \times 3''$)
- Digital spectrometers: Ortec DigiBase and Xia Polaris



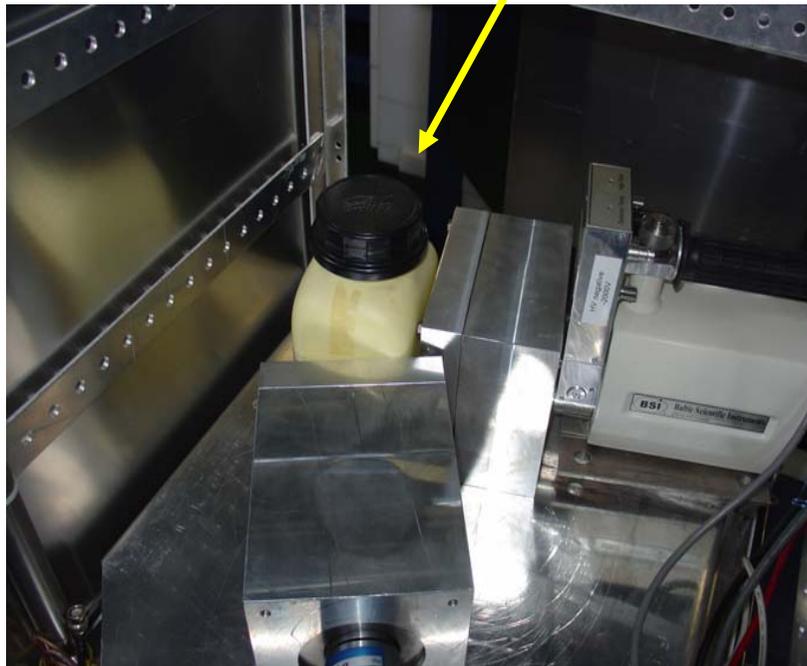
Shielding arrangements for LaBr_3 scintillation detectors





Experimental setup

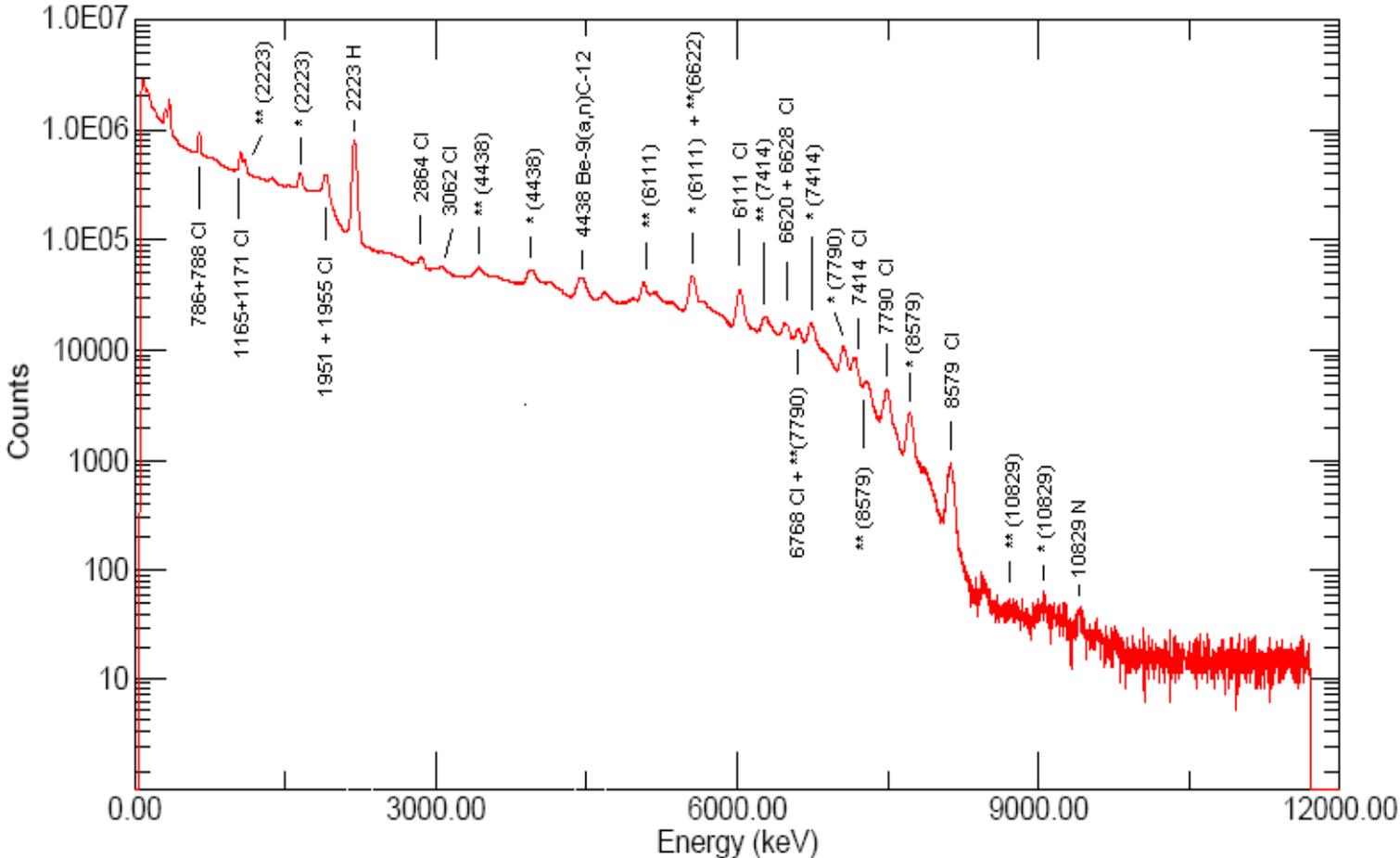
Test sample (NH_4Cl)



Neutron generator



Gamma spectrum, PGNAA neutron capture



NH₄Cl sample by small LaBr₃ scintillation detector



Result, example: element ratios

Test sample: NH_4Cl , and evaluation of the mass ratios H/Cl

Method: prompt thermal activation analysis

Measured mass ratio H/Cl	Theoretical mass ratio H/Cl
0.10 ± 0.03	0.11



Objective: Detection of Special Nuclear Material for Nuclear Security applications

Application:

- detection of SNM in shielded containers

Physics principle:

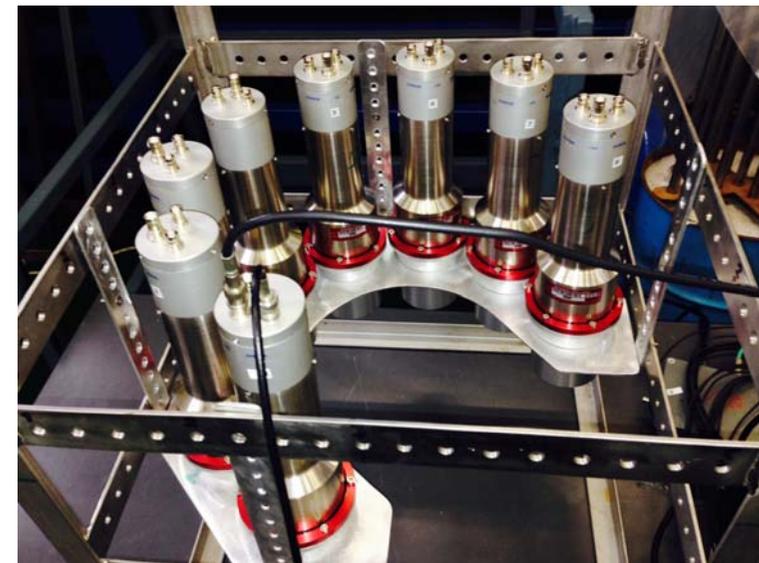
- induce fission by epi-thermal/thermal neutrons (pulsed neutron source)
- fission signatures are the evidence for presence of SNM:
 - only fast prompt fission neutrons appear in PSD peak
 - neutron coincidences in short gates of 10-20 ns.

Technical implementation issues:

- high count rates in neutron/photon mixed fields
- fast neutron detection at high efficiency
- good neutron/photon separation necessary

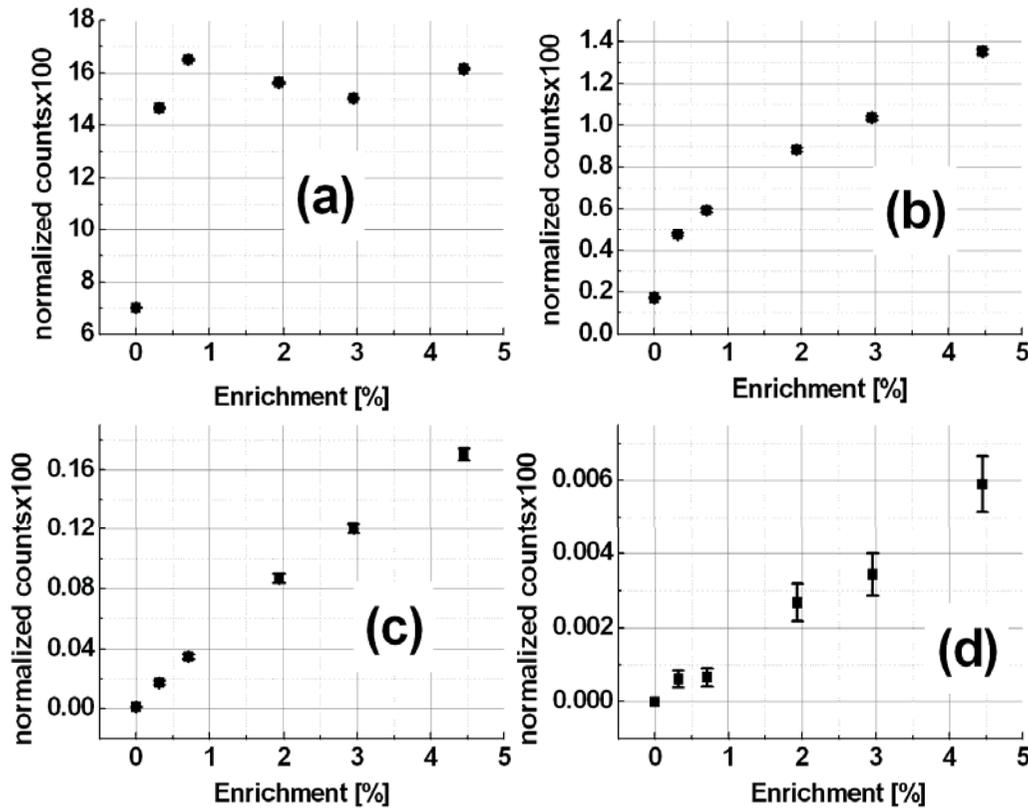
Technical/scientific implementation:

- 8x 3"x3" EJ-309 liquid scintillation detectors
- signal analyzing hardware (fast digitizers)
- (online interpretation (PSD) in FPGA hardware)





Detection of triple coincidences using CBNM U_3O_8 standards thermal interrogation ($250\mu s$ to $40000\mu s$)



Kinds of three-fold coincidences:

- (a) $\gamma\gamma\gamma$
- (b) $\gamma\gamma n$
- (c) $\gamma n n$
- (d) $n n n$



Example I, nuclear security:

Scale-up of experimental results to industrial size (preliminary results)

PUNITA simulations:

- To compare experiments to MCNP model we apply the following figure of merit:
 $FOM = [\text{thermal n-flux}] \times [\text{detector n-efficiency}]$
in order to optimize both parameters individually.

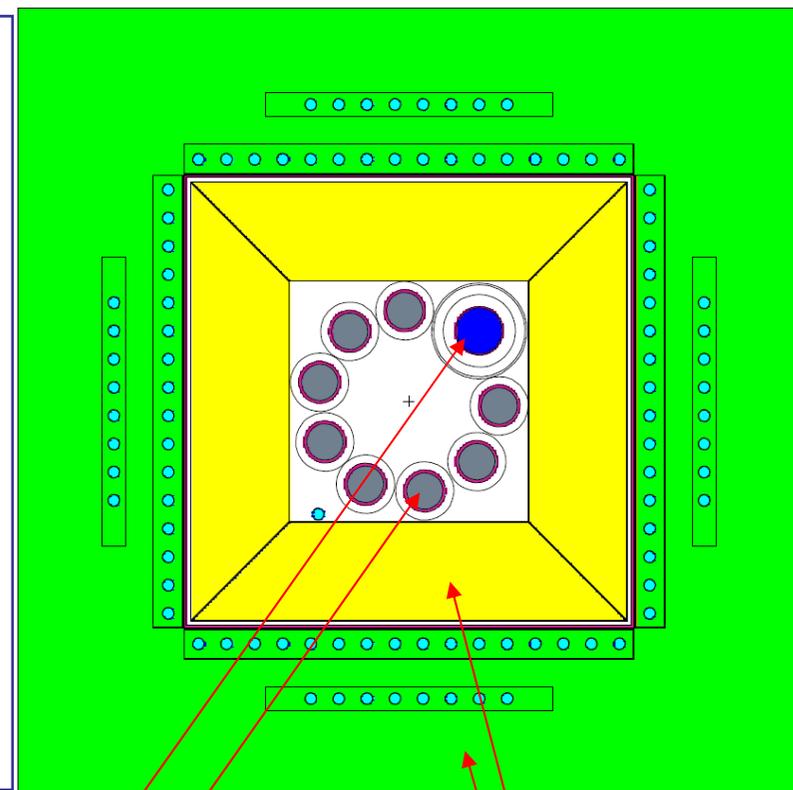
- Due to generator pulsing (100 Hz) we integrate n-flux over 10 ms period:

$$\varphi_{th} = 224891 \text{ cm}^{-2}$$

- As a measure of fission neutron detection efficiency we define recoil protons with $E_{kin} > 700 \text{ keV}$ as a neutron detection event:

$$\varepsilon_n = 6.07 \pm 0.30 \%$$

MCNP model of PUNITA setup



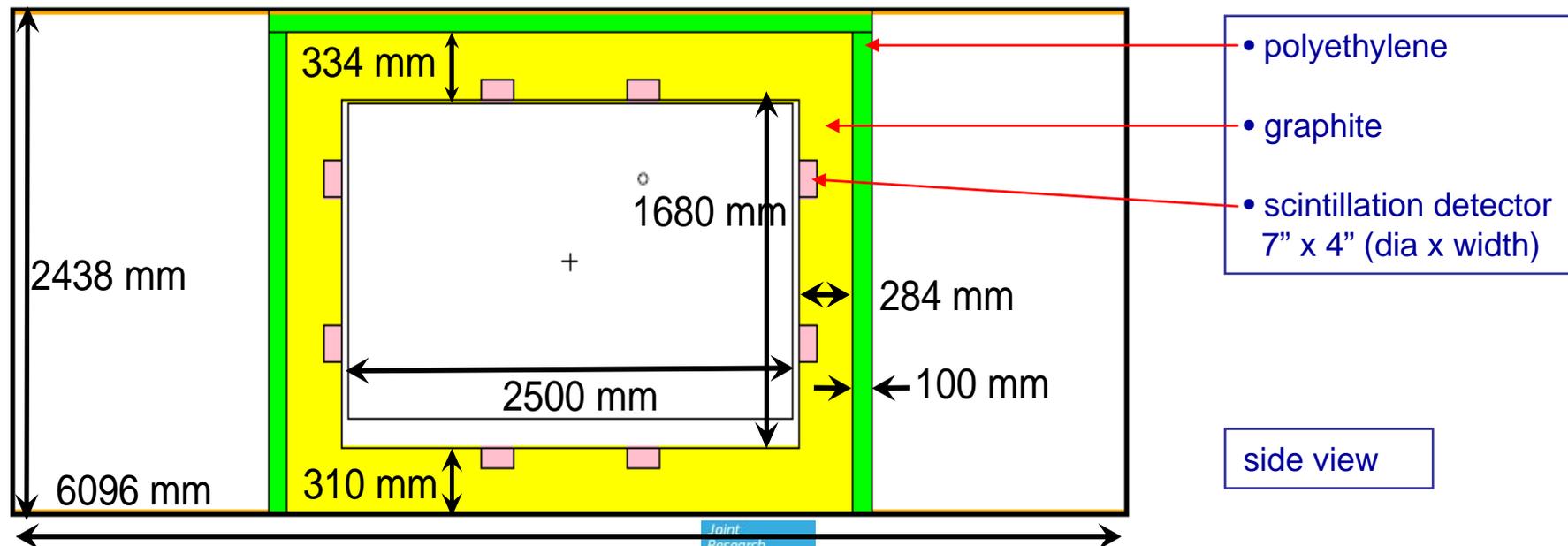
- neutron generator
- graphite
- scintillation detector
- polyethylene

Detection of SNM



Scale-up of experimental results to industrial size (preliminary results)

- Assay of Unit Load Devices (ULDs) for air cargo
- Accommodate largest standard ULD:
LD1 for Boeing 747-400: 234 x 153 x 163 cm, 4.90 m³
- Device implemented into standard 20-foot container
not shown: - conveyer belt through entire container
- n-generator centered below cavity

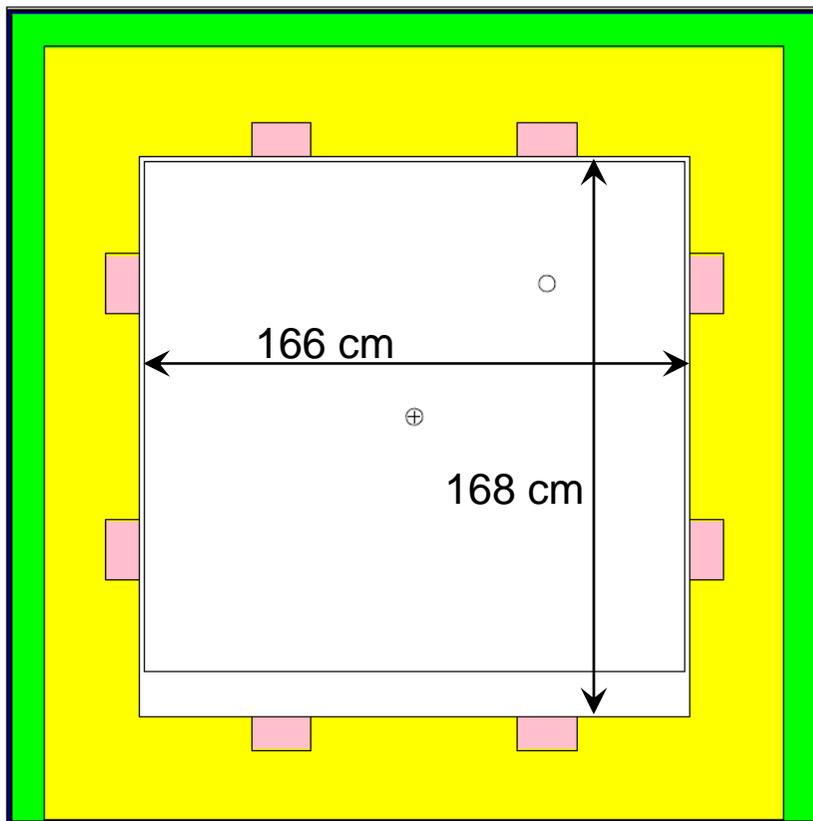


Detection of SNM



Scale-up of experimental results to industrial size (preliminary results)

Assay device as implemented in 20-foot container
(view from entrance) showing:
detector positions, n-generator position



Standard ULD as applied for air cargo





Scale-up of experimental results to industrial size (preliminary results)

Geometry comparison “ULD device / PUNITA”:

moderator volume ratio	11.9	
• sample cavity size ratio	31.7	
• detector volume ratio	21.8	
• neutron generator	same	(1×10^8 /sec, 100 Hz pulsing)

Preliminary simulation results “ULD device / PUNITA”:

thermal n-flux ratio, centre pos.	$\Phi_{th, ULD \text{ device}} / \Phi_{th, PUNITA} = 0.0061 \pm 0.0002$
fission neutron det. efficiency	$\epsilon_{n, ULD \text{ device}} / \epsilon_{n, PUNITA} = 0.379 \pm 9.94 \cdot 10^{-4}$
FOM ratio	$1/436 \pm 1/11065$

Based on the (conservative) estimate of a detection limit in the PUNITA configuration of 0.52 g ^{235}U in a 100 second measurement when using the 3-fold neutron coincidences as signature, the detection limit of the “ULD device” described above would be approximately a factor 436 higher, or LOD = 228 g ^{235}U .

Pulsed neutron interrogation for nuclear disarmament verification

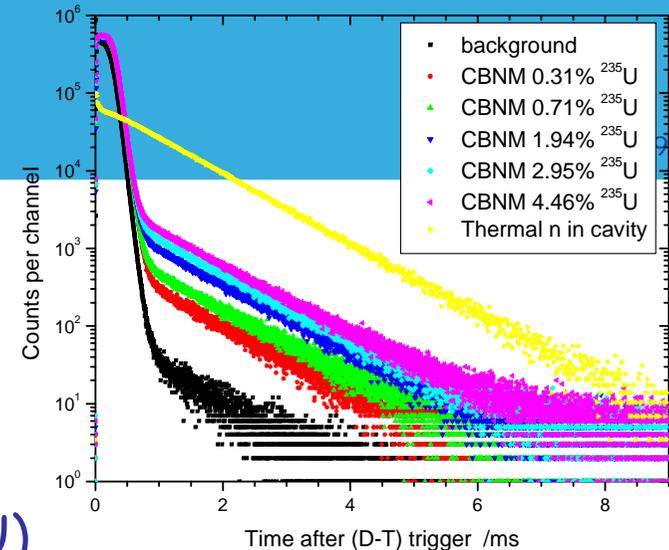
Verification of nuclear materials (Pu and HEU)

Neutron generator (OK), moderator materials (probably not OK)

- Pulsed n-generator (14-MeV)
- Tailor n-spectrum with filters around target
- Induce fission with fast and epi-thermal spectrum
- Detect useful fission signatures, in particular prompt fission neutrons
- Use die-away curve (neutron self-multiplication) to confirm "presence of large fissile mass".

Verification of high explosives

- Need for moderator to acquire useful neutron capture gamma lines
- In-elastic scattering gamma lines are difficult



Thank you!



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