

LA-UR-17-29028

Approved for public release; distribution is unlimited.

Title: Nondestructive Assay for International Safeguards: Context, Science,
and Technology

Author(s): Trahan, Alexis Chanel

Intended for: Nuclear Science and Security Consortium live webinar

Issued: 2017-10-03

Disclaimer:

Los Alamos National Laboratory, an affirmative action/equal opportunity employer, is operated by the Los Alamos National Security, LLC for the National Nuclear Security Administration of the U.S. Department of Energy under contract DE-AC52-06NA25396. By approving this article, the publisher recognizes that the U.S. Government retains nonexclusive, royalty-free license to publish or reproduce the published form of this contribution, or to allow others to do so, for U.S. Government purposes. Los Alamos National Laboratory requests that the publisher identify this article as work performed under the auspices of the U.S. Department of Energy. Los Alamos National Laboratory strongly supports academic freedom and a researcher's right to publish; as an institution, however, the Laboratory does not endorse the viewpoint of a publication or guarantee its technical correctness.

Nondestructive Assay for International Safeguards

Context, Science, and Technology



Alexis Trahan, Ph.D.

October 23, 2017

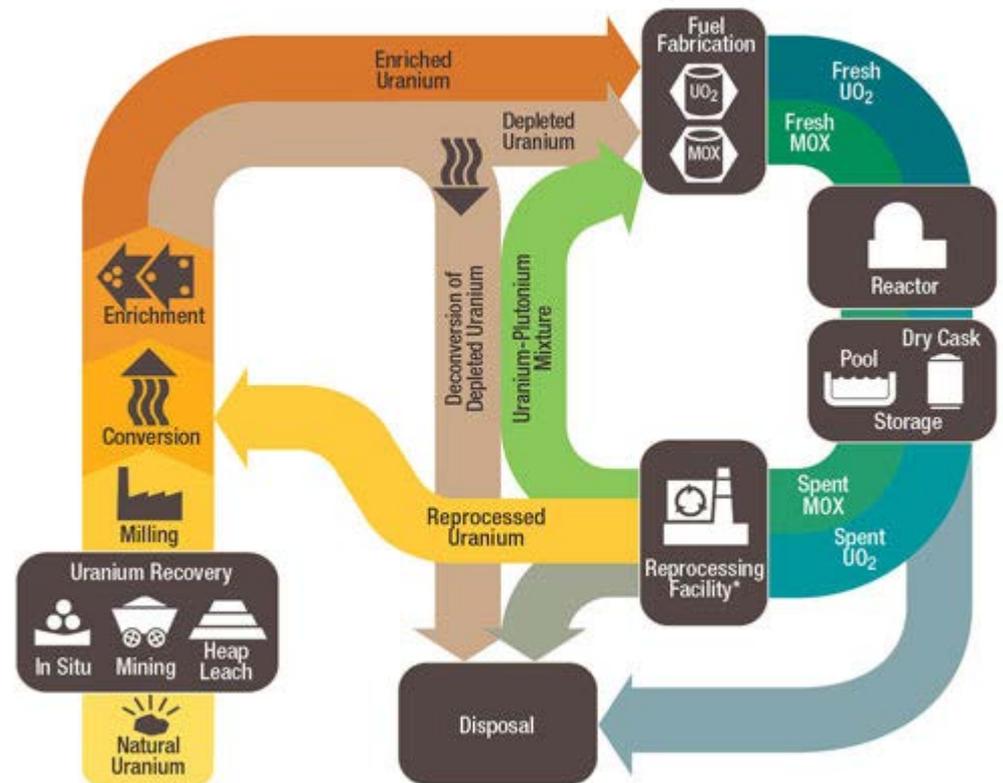


Operated by Los Alamos National Security, LLC for the U.S. Department of Energy's NNSA

International Safeguards

- “Delivering Effective Nuclear Verification for World Peace”
- The objective of safeguards is the timely detection of diversion of significant quantities of nuclear material from peaceful nuclear activities to the manufacture of nuclear weapons
- Inspect nuclear facilities worldwide, monitor amounts of nuclear materials to ensure that it isn't going to illicit uses

The Nuclear Fuel Cycle



* Reprocessing of spent nuclear fuel including MOX is not practiced in the U.S.
Note: The NRC has no regulatory role in mining uranium.

NRC.gov

The IAEA Today

Currently, the IAEA is working to achieve....

- **Universal acceptance of the AP**
- **Integrated safeguards**
 - Non-discriminatory and tailored to specific facility types
- **Safeguards-by-design**
 - Integrated within a facility's design, covering safeguards and security
- **Unattended monitoring & data integration**
 - Robust data management systems to reduce on-site inspector presence
- **State-level concept/approach**
 - Assessing each State as a whole

Source(s):

<http://www.iaea.org/safeguards/statements-repository/overview.html>

http://www.iaea.org/safeguards/documents/LongTerm_Strategic_Plan_%2820122023%29-Summary.pdf

AT THE END OF 2015:

200,110 Significant quantities* of nuclear material were under IAEA safeguards

1,286 Nuclear facilities and locations outside facilities were under IAEA safeguards

2,118 In-field inspections conducted

623 Design information verifications were conducted

64 Complementary accesses were conducted



182 IAEA Safeguard Agreements States

128 Additional Protocols in force States

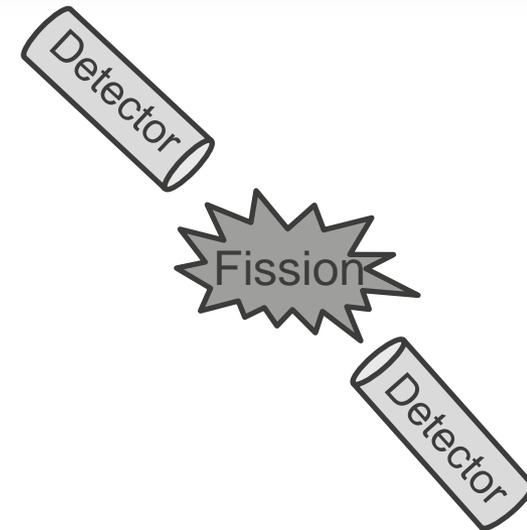
IAEA (2016). IAEA Safeguards 2016: Serving Nuclear Non-Proliferation.

* One significant quantity is the approximate amount of nuclear material for which the possibility of manufacturing a nuclear explosive device cannot be excluded.

Special Nuclear Materials

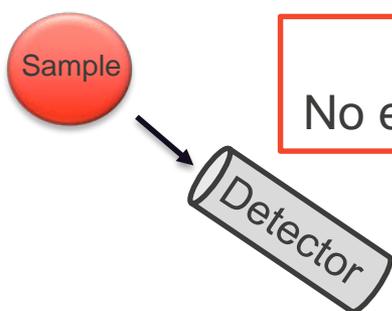
- **What are we trying to verify?**
 - Special nuclear material (SNM) is where it should be, and in the proper amount

- **SNM: Nuclear materials that can be used to make a weapon**
 - Highly Enriched Uranium (HEU)
 - Diversion path for HEU: enrichment facilities
 - Certain isotopes of U undergo fission primarily when induced with a neutron source
 - Weapons Grade Plutonium (Pu)
 - Diversion path for Pu: spent fuel (repositories, interim storage, reprocessing facilities)
 - Certain isotopes of Pu undergo fission spontaneously, without any prompting

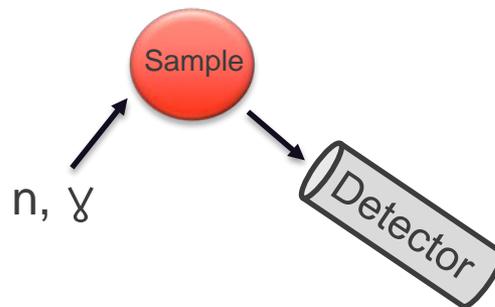


Nondestructive Assay (NDA)

- NDA is the most commonly employed technique for material accountancy
- A series of gamma or neutron detectors are typically used to measure radiation emitted from the sample of interest
- Energy, timing, and intensity of radiation may be correlated to isotope type and quantity in the sample



Passive:
No external source



Active:
Neutrons or gammas irradiate source to magnify signal

- Passive interrogation requires good signal intrinsic to sample (^{240}Pu , ^{252}Cf)
- Active interrogation requires fissile material or material prime for gamma interactions (^{235}U , ^{239}Pu)



Rail radiation portal monitor (RPM) at the Port of Antwerp, Belgium

Neutrons and Photons



- Spontaneous and induced fission
 - (α, n)
 - Cosmic rays
 - (p, n)
 - $(n, 2n)$
 - (γ, n)
- } Less common

- Nucleus (gamma-ray)
- Nuclear collision (gamma-ray)
- Electron cloud (x-ray)

Time and correlations

Energy

Low Z material

High Z material

^3He , Scintillators, fission chambers

HPGe, Scintillators, NaI

Origins
Signal
Shielding
Detectors

Neutrons

History of Neutron Counting

- **TOTAL NEUTRON**

- Record the total number of neutrons detected in a certain amount of time
- Accurate assays can be obtained only for very few types of SNM

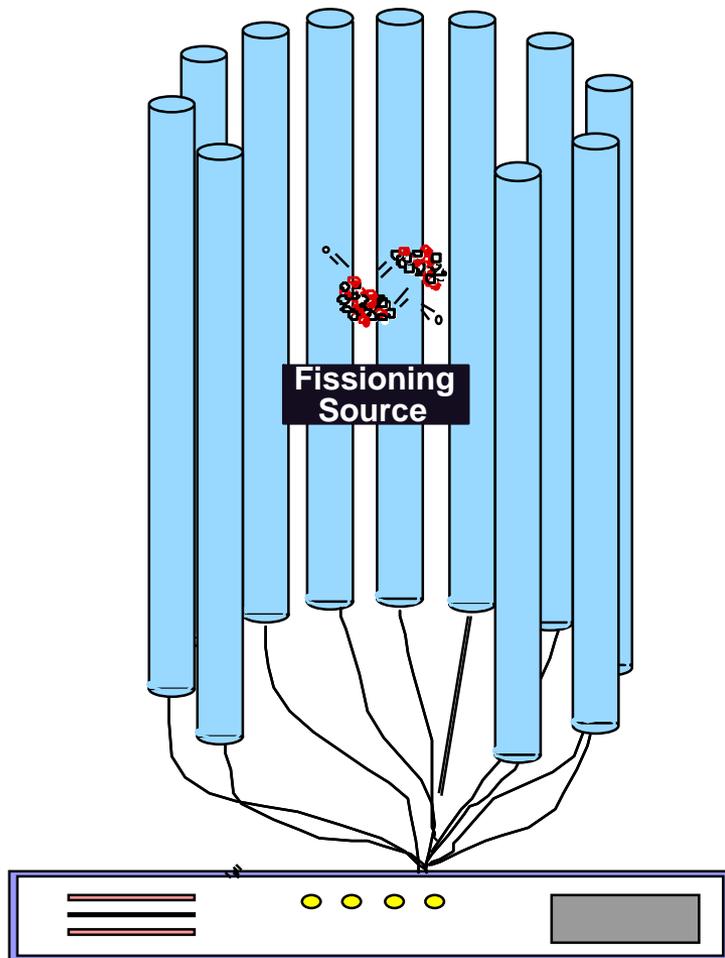
- **COINCIDENCE COUNTING**

- Record the number of times two neutrons arrive within a set time window (gate)
- Wide application for international safeguards
 - focused on verifying declared materials

- **NEUTRON MULTIPLICITY COUNTING**

- Extension of neutron coincidence counting
- Record the number of times we detect 2, 3, 4, etc. neutrons within a gate
- It improves neutron assay accuracy dramatically by adding more measured information

Passive Neutron Counter



^3He neutron detectors

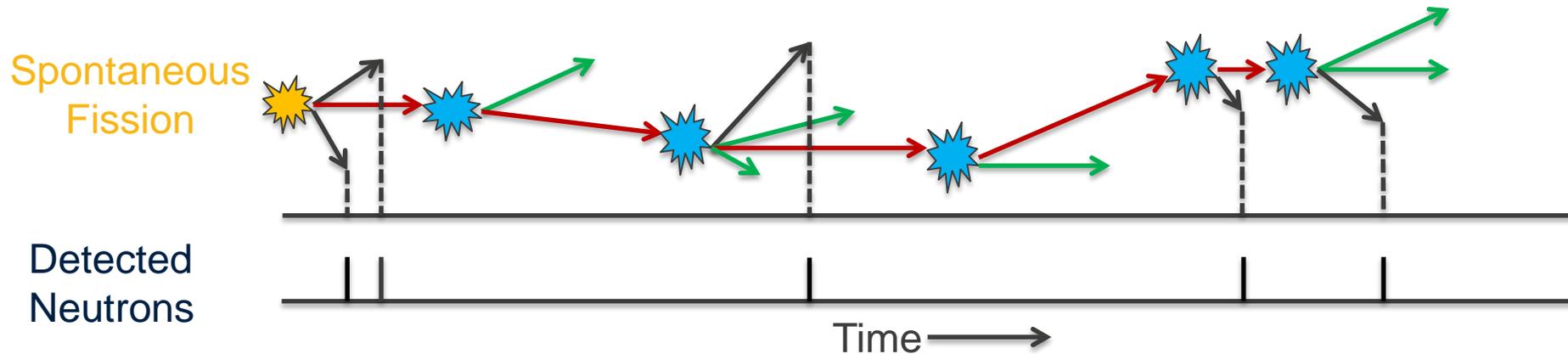
Fissioning source surrounded by neutron detectors

Emission of multiple **prompt** neutrons from fission detected as coincident neutron events

Multiplicity information is used to calculate the mass of fissioning isotopes

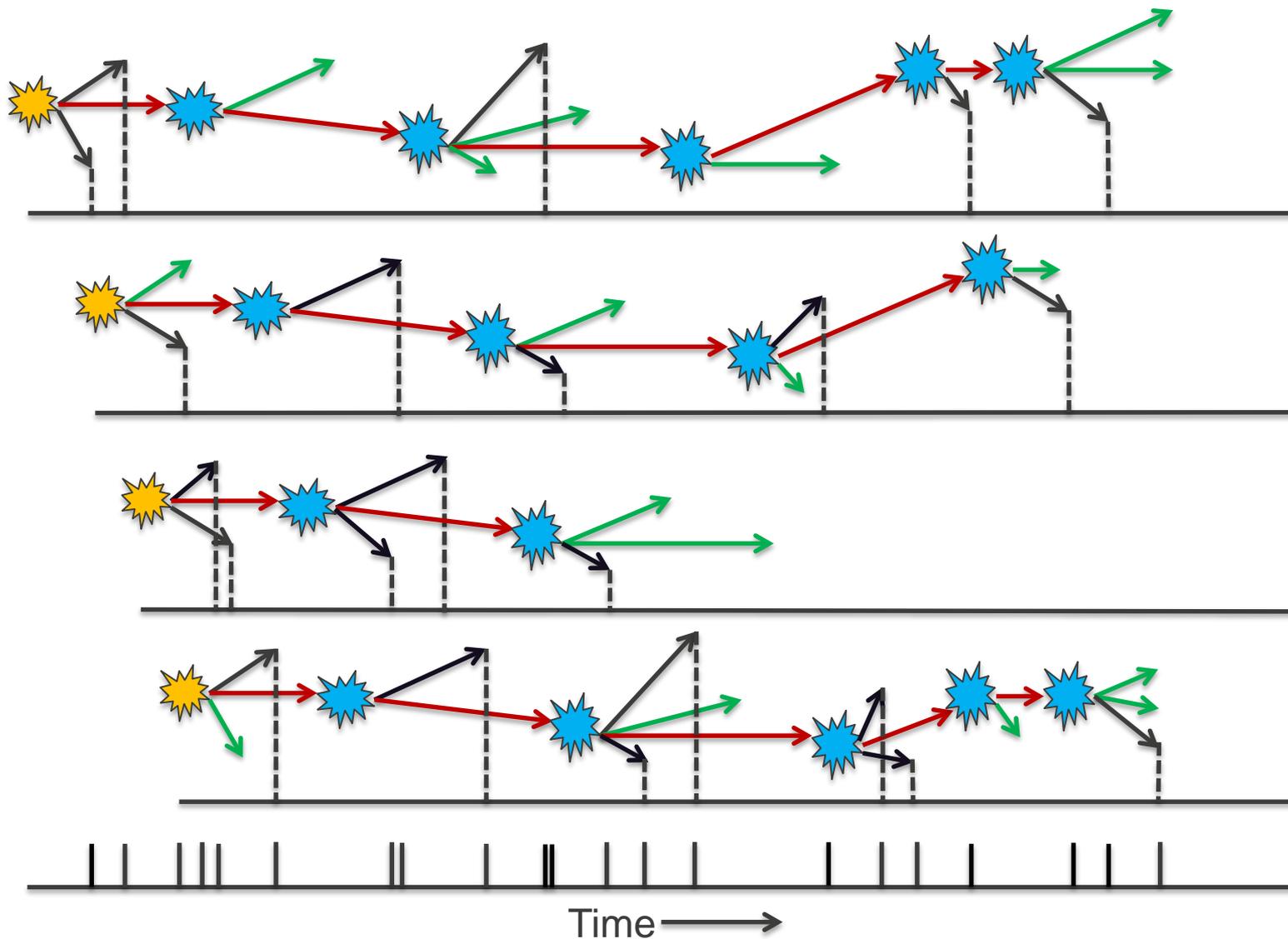
Pulse-processing Electronics

Neutron Coincidence Counting

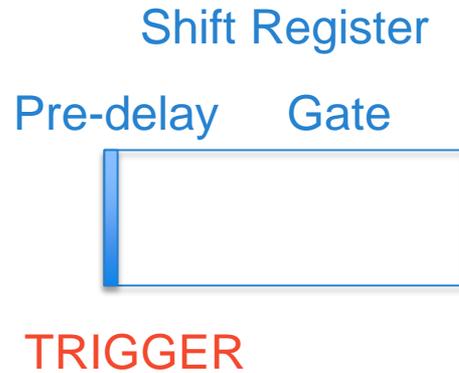


- As neutrons are detected, they trigger the shift register and neutron coincidence counting is performed

Neutron Coincidence Counting



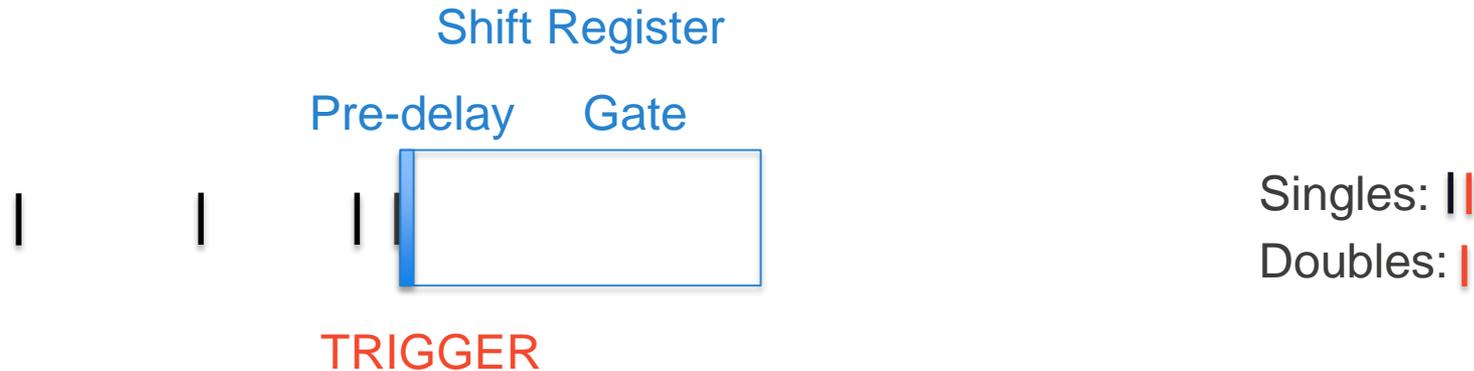
Neutron Coincidence Counting



Singles: |
Doubles:

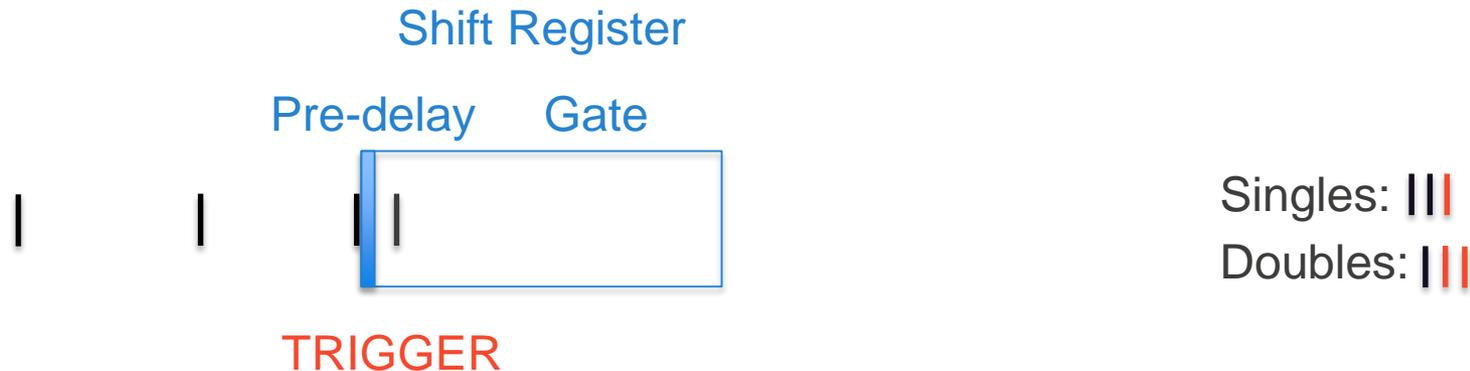
- First neutron detection triggers the shift register

Neutron Coincidence Counting



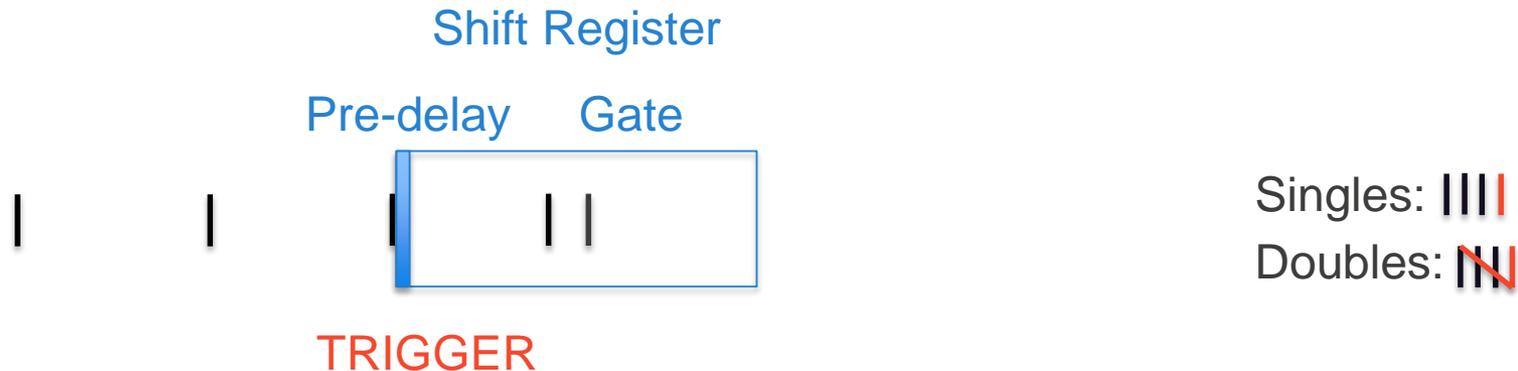
- **First neutron detection triggers the shift register**
- **Next neutron detection triggers the shift register again; this time, one neutron is already in the shift register, so we have one coincidence**

Neutron Coincidence Counting



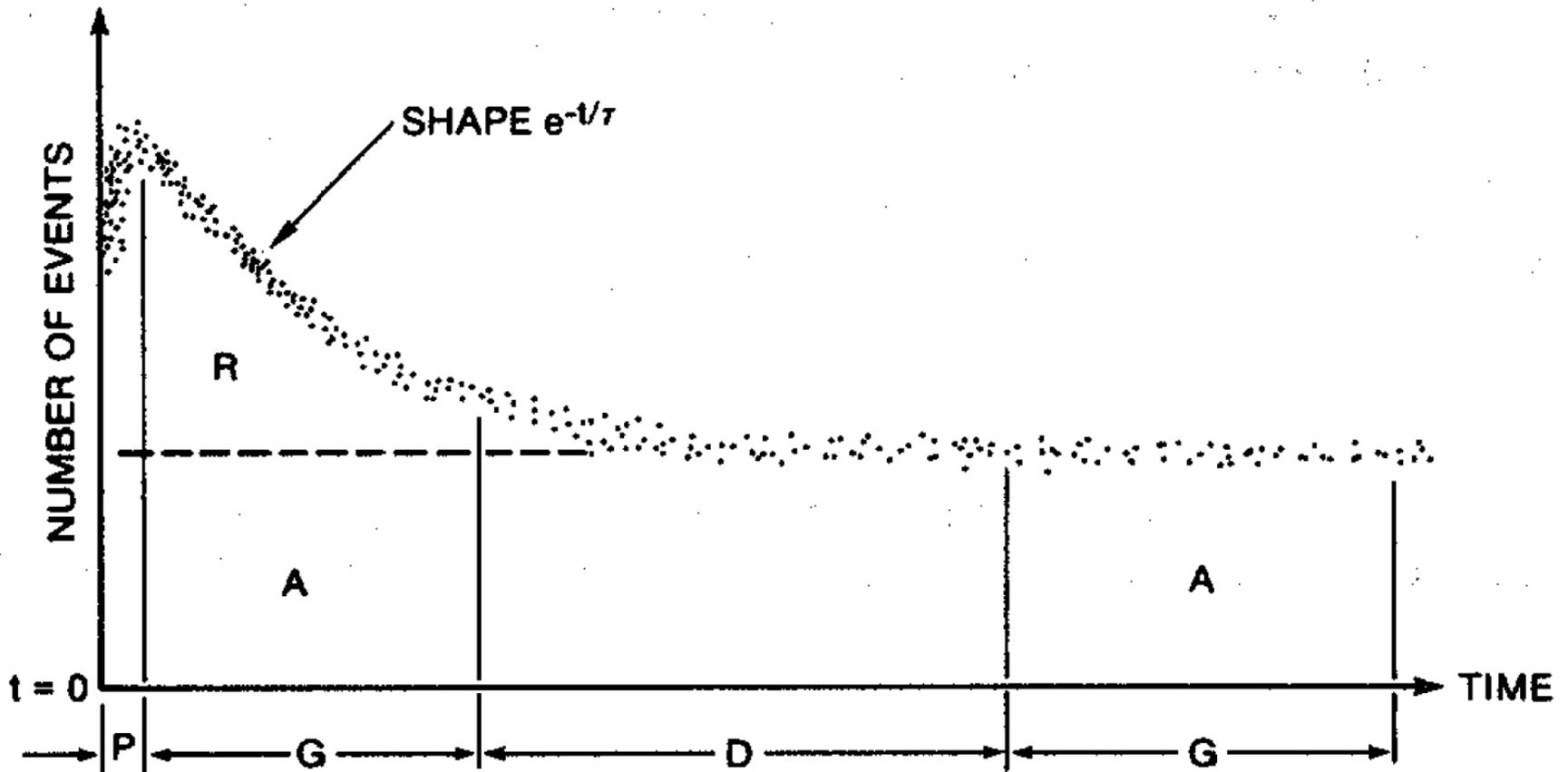
- First neutron detection triggers the shift register
- Next neutron detection triggers the shift register again; this time, one neutron is already in the shift register, so we have one coincidence
- Next neutron detection triggers the shift register again; this time, **two neutrons** are already in the shift register, so we have **two coincidences**

Neutron Coincidence Counting

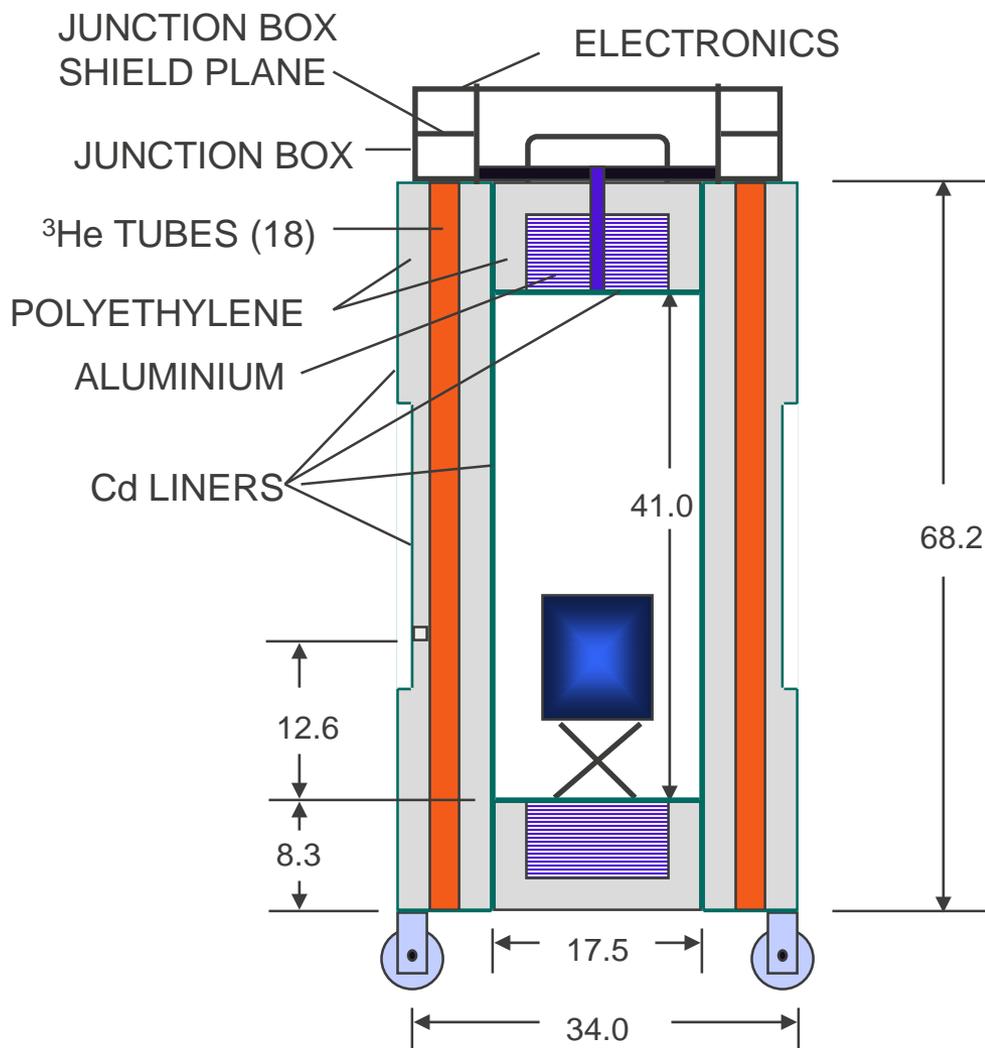


- First neutron detection triggers the shift register
- Next neutron detection triggers the shift register again; this time, one neutron is already in the shift register, so we have one coincidence
- Next neutron detection triggers the shift register again; this time, **two neutrons** are already in the shift register, so we have **two coincidences**
- Next neutron detection triggers the shift register again; this time, **two neutrons** are already in the shift register, so we have **two coincidences**

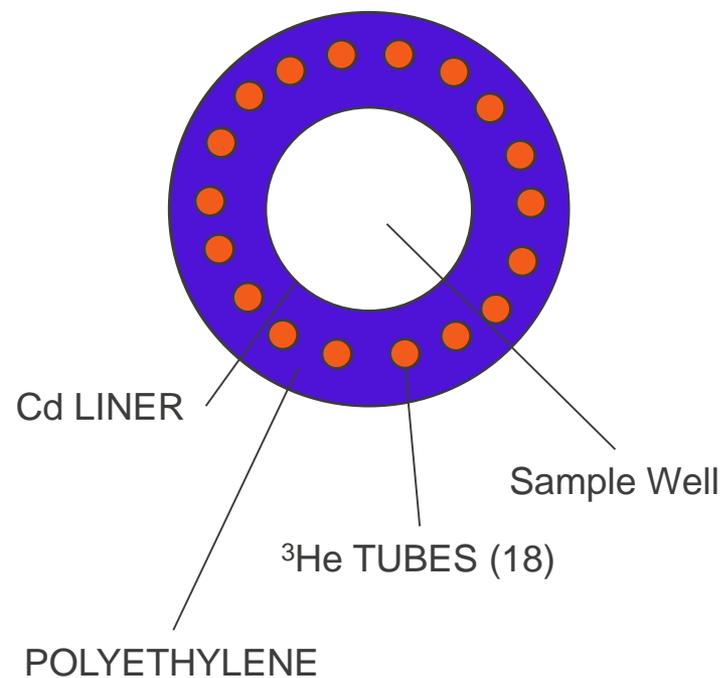
Rossi-Alpha Distribution



High-Level Neutron Coincidence Counter (HLNC)

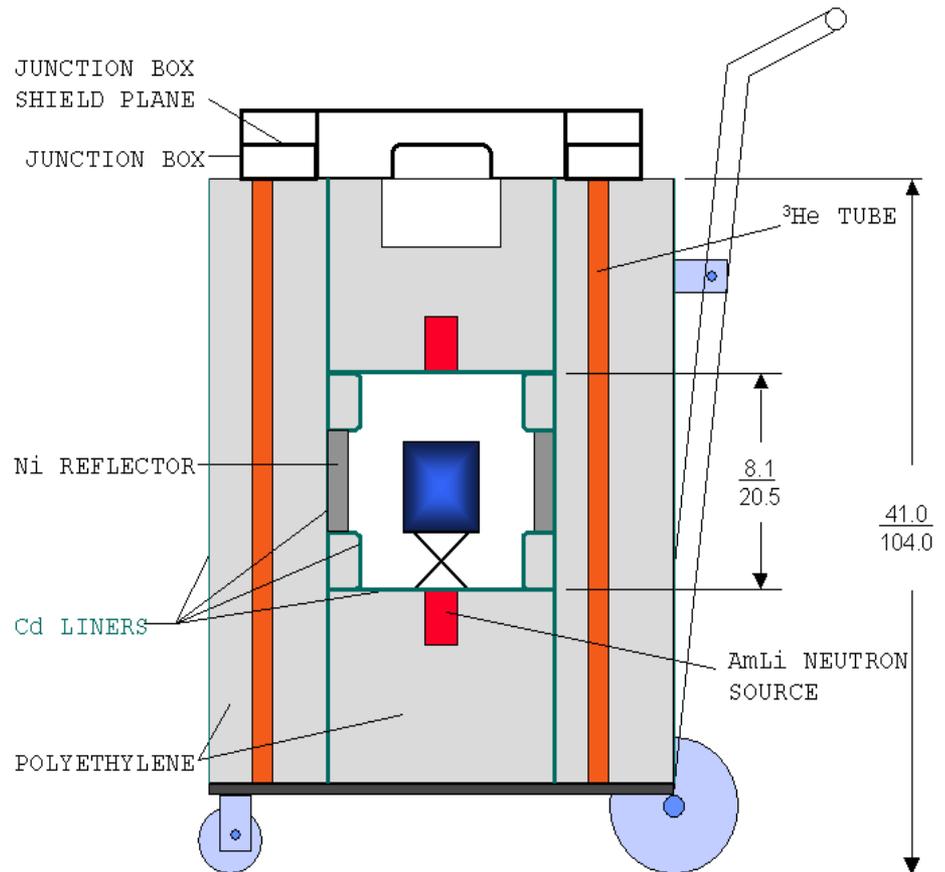
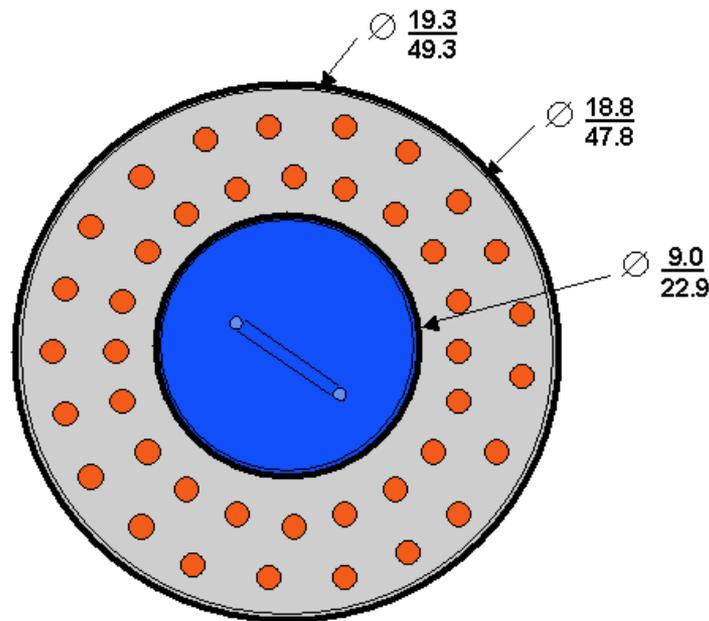


- $\varepsilon = 17.5\%$
- $\tau = 43 \mu\text{s}$
- 18 detector tubes: 4-atm ^3He , 50.8-cm active-length, $\phi 2.54\text{cm}$
- 6 Amptek preamplifiers



Active Well Coincidence Counter (AWCC)

- $\varepsilon = 33\%$
- $\tau = 51 \mu\text{sec}$
- 42 tubes: 4-atm ^3He , 50.8-cm active-length, $\phi 2.54\text{cm}$
- 6 Amptek preamplifiers



Photons

Photons

- **Generic Assay Equation**

$$M_{SNM} = \frac{R_{Rad} \times CF}{Cal}$$

M_{SNM} = Mass of special nuclear material

R_{Rad} = Measured radiation rate (counts per unit time) from SNM item

CF = Correction for losses due to:

- item self absorption
- container absorption
- measurement system electronics

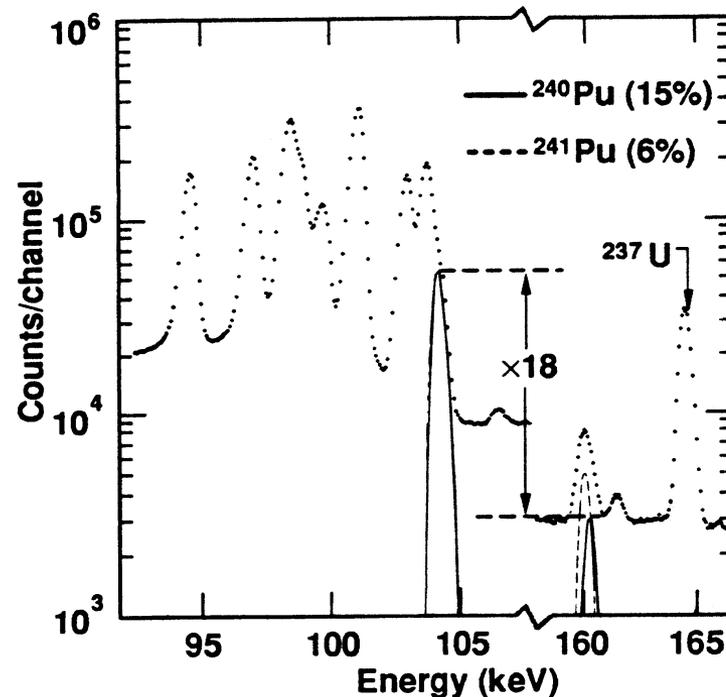
Cal = Calibration constant

MGA

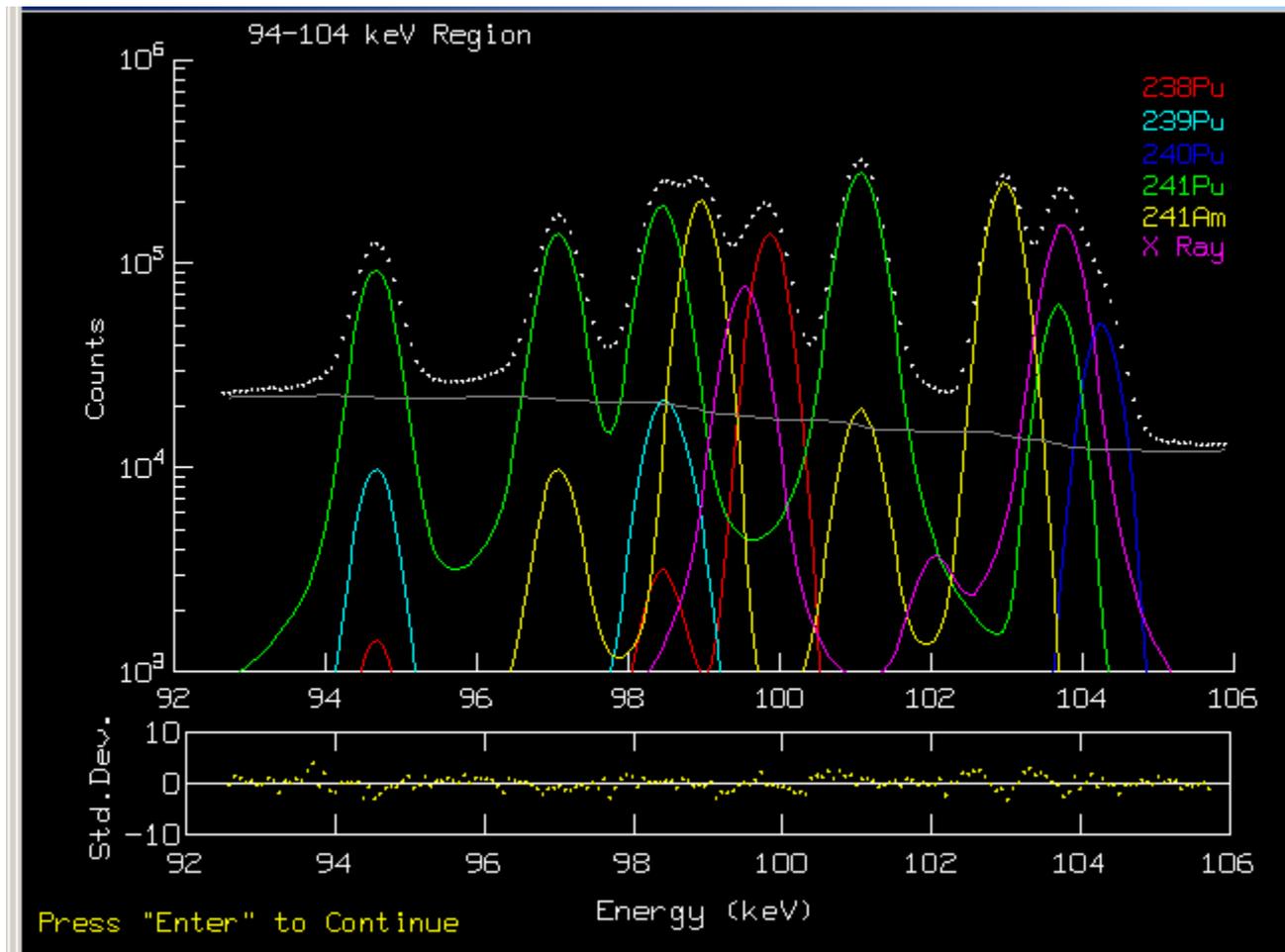
- Multi-Group Analysis of 100 keV x-ray and gamma-ray region
- Plutonium isotopics determination
- Uses multiple peaks to create an efficiency curve, eliminating the need for a separate efficiency measurement
- High-resolution Ge detector needed

^{240}Pu Peak Intensity:

Compare the intensity of the peak at 104 keV with that of the one at 160 keV



MGA



```

Report generated 1-Feb-2013 09:18:33
Spec. ID: CBNM61L.CNF                               LT: 38.9 Mins DT:44%
Measurement Date: 8-Sep-1992 Declared Date: 8-Sep-1992
Sample ID:

PU g/cm2 = 2.7056   CD thickness = 1.35 mm           FWHM at 122 keV = 531 eV
QFIT = 1.60        Gain = 75.154 eV/ch              at 208 keV = 663 eV
NQFIT= 1.007
ISOTOPIC ANALYSIS AT
  RELATIVE  %   %*   MEAS. DATE   DECLARED DATE   SPECIFIC POWER
  ABUNDANCE ERR  ERR   WT.PCT.  %ERR   WT.PCT.  %ERR (MILLIWATTS/GM)
Pu238 = 0.018208 0.2 0.7   1.16741 0.65   1.16741 0.65   6.62589
Pu239 = 1.000000 0.6 0.0   64.11562 0.51   64.11562 0.51   1.23666
Pu240 = 0.407647 0.5 0.8   26.13654 0.69   26.13654 0.69   1.85109
Pu241 = 0.078905 0.1 0.7   5.05902 0.62   5.05902 0.62   0.17261
Pu242 = (Default Algorithm) 3.5214 (10) 3.5214 (10) 0.00408
Am241 = 0.050407 0.2 0.6   3.23185 0.61   3.23185 0.61   3.69077
      *=Error in Ratio (1 Sigma Error) TOTAL= 13.581 +/-0.51%
241Am Separated About 10.227 +/-0.069 Years before Measurement.

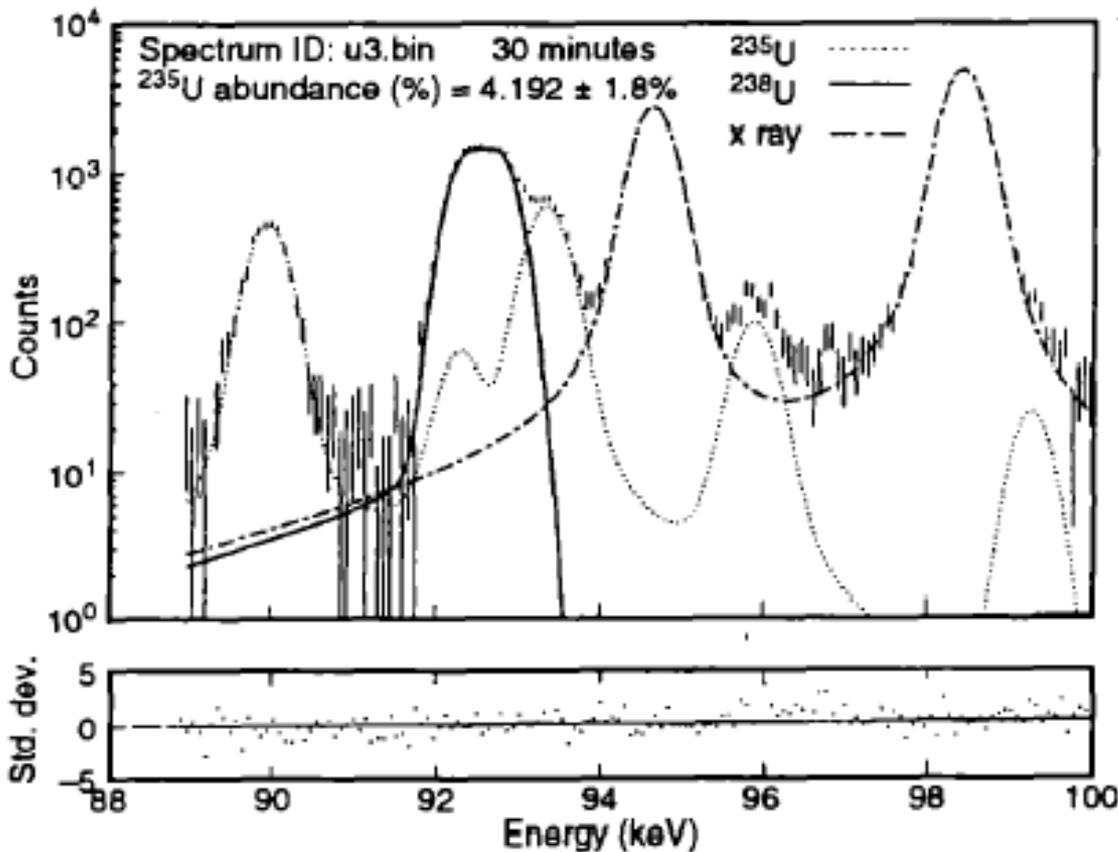
Pu-240 effective = 34.99 (at meas. date) 34.99(at Decl. date) +/- 1.83%

```

Press "Enter" to Continue

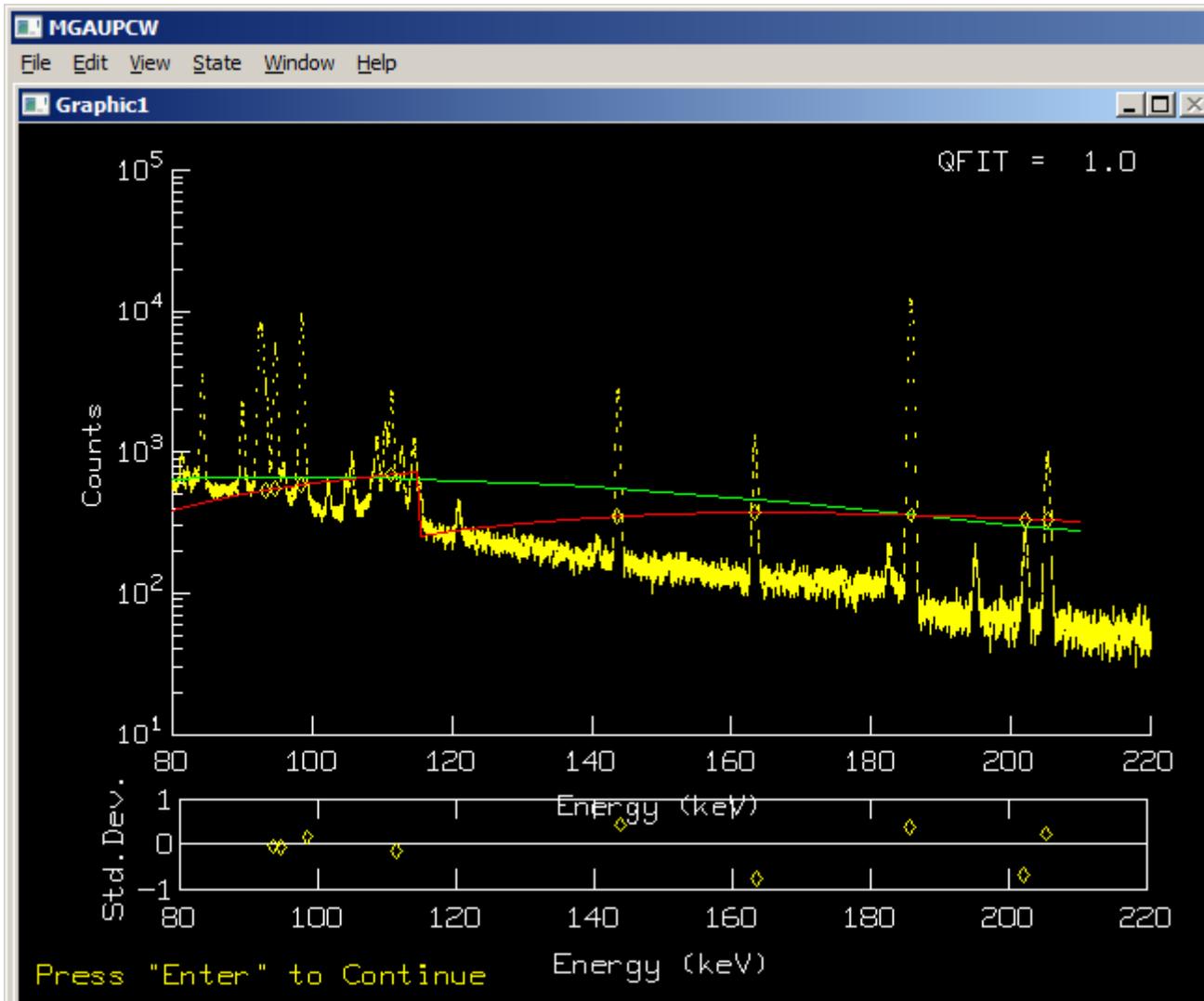
MGAU

- Multi-Group Analysis for Uranium
- Uranium enrichment measurement method
 - Uses 89-120 keV region of uranium gamma-ray spectrum

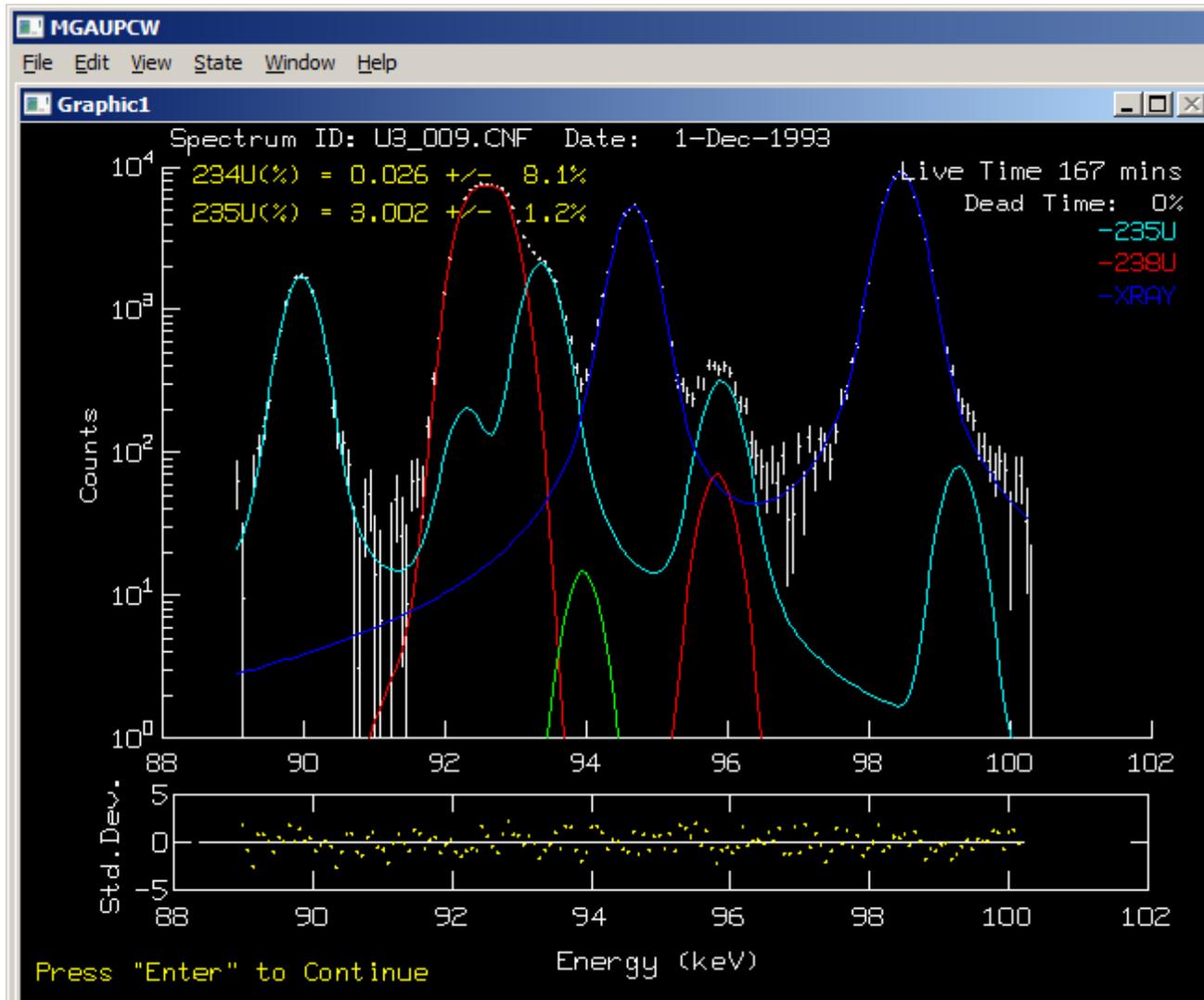


- The low energy region of the spectrum is very complex with many overlapping X-ray and gamma ray peaks

MGAU



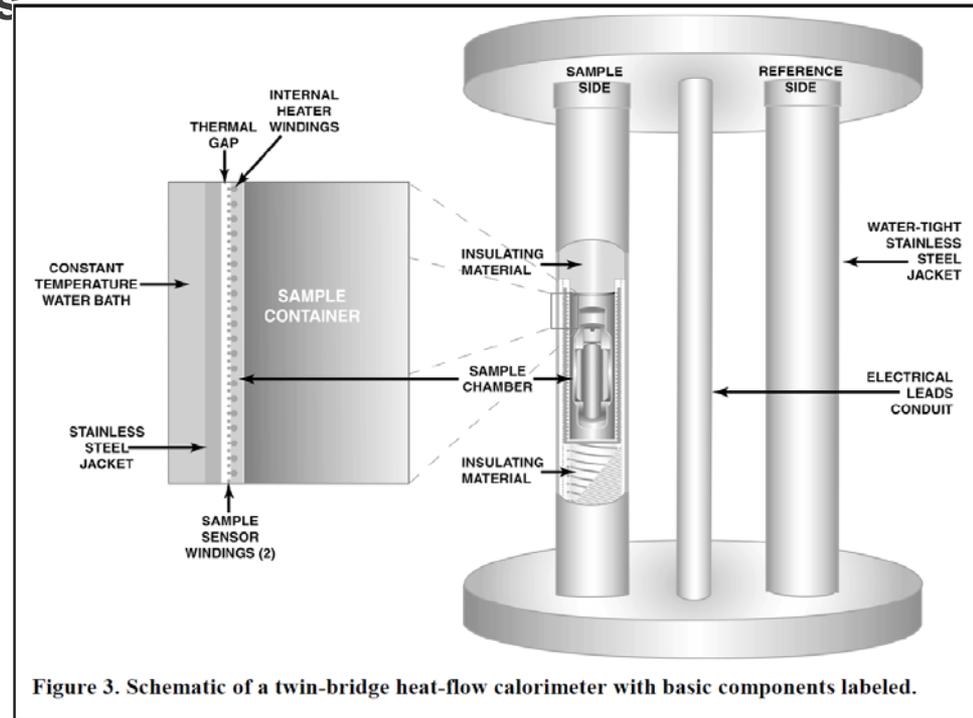
MGAU



Heat

Calorimetry

- Well-established, precise method of NDA
- Uses thermal power generated by radioactive decay in the sample to determine the mass of special nuclear material
- Heat flow calorimetry is most commonly used for safeguards NDA
- 60 Wheatstone bridge calorimeters currently being used for Pu and tritium measurements at LANL
- Bulk measurements can be taken without issues from absorption or self-shielding
- Takes much longer than other NDA techniques



Spent Fuel NDA

Spent Fuel NDA: Objectives

- **Verify operator declaration of residual uranium, and buildup of plutonium**
 - Burnup
 - Initial enrichment
- **Verify cooling time of assembly to assist with other parameters**
- **Verify completeness of assemblies**

- **IAEA R&D plan lists a milestone requesting “...more sensitive and less intrusive alternates to existing nondestructive assay (NDA) instruments”**
 - IAEA Department of Safeguards Long-Term R&D Plan, 2012-2023, Vienna, January (2013).

Spent Fuel NDA: Challenges

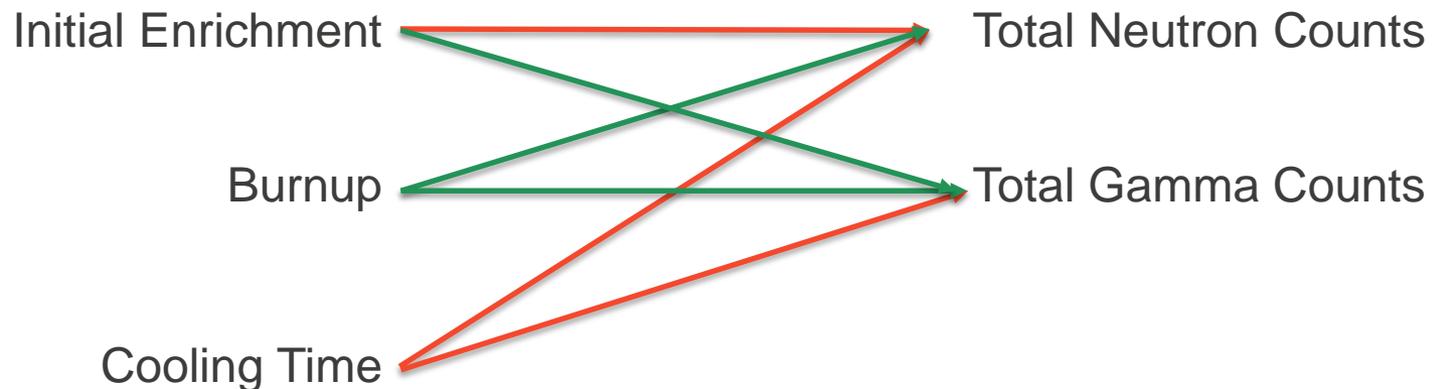
- **Interruptions to reactor operations**

- Nuclear facilities have a standard way of operating and large disruptions (i.e. long measurements, drastic fuel movement) are not acceptable

- **Fuel inhomogeneity**

- Both axially and radially, neutron flux in the reactor affects burnup, resulting in inhomogeneous fuel assemblies

- **Competing parameters**

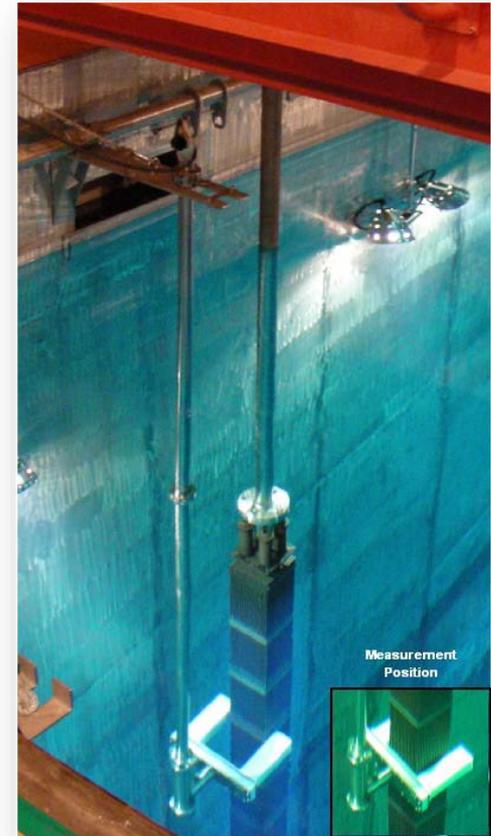


- **Very difficult to accurately model**

- Burnup codes are highly dependent on the accuracy of nuclear data and reactor operating history

Fork Detector

- **NDA technique widely used by the IAEA and EURATOM**
- **Detector system straddles light water reactor fuel assemblies with four fission chambers (neutrons) and two ion chambers (gammas)**
 - Total gamma and neutron intensities as well as ratios of intensities give information about fuel assembly such as cooling time and burnup
 - One of the fission chambers is wrapped in cadmium to provide a means for estimating multiplication
- **Other versions of the Fork detector exist with ^3He tubes instead of fission chambers, etc.**



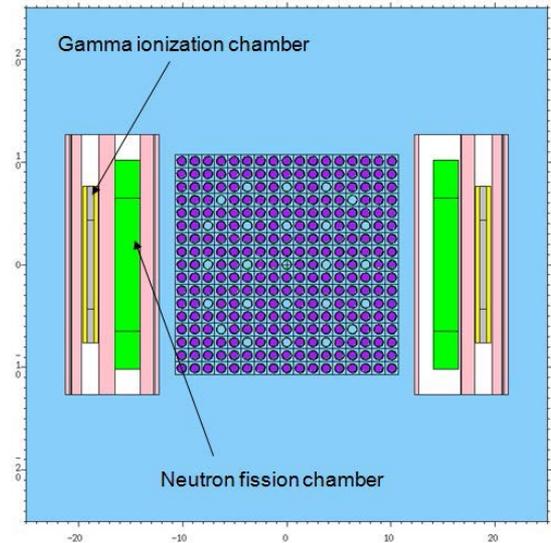
Fork Detector

- **Benefits**

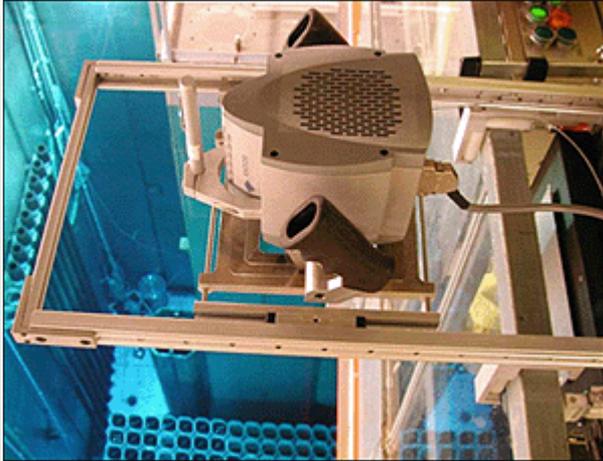
- Rugged, reliable, validated and verified, easy to use
- Requires minimal fuel movement

- **Drawbacks**

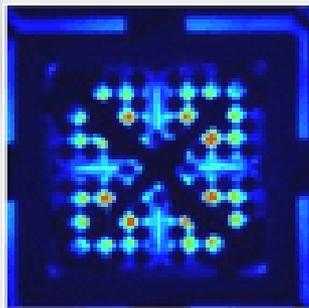
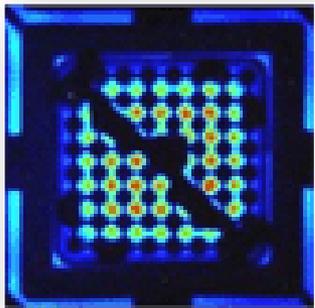
- Asymmetric burnup could affect gamma signal
- Assumptions about how neutron and gamma counts trend with burnup and cooling time fall apart under irregular burning history
- Results rely heavily upon data provided by operator
- May not be able to detect pin removal under 50%



Cerenkov Viewing Device



- Viewing device sensitive to ultraviolet radiation in the water surrounding spent fuel
- Cerenkov radiation provides the UV light and is derived from the intense gamma radiation in spent fuel
- Electrons may exceed the speed of light in water and therefore must lose energy by emitting Cerenkov radiation. β particles contribute as well
- Glow patterns above fuel rods used to distinguish fuel from non-fuel



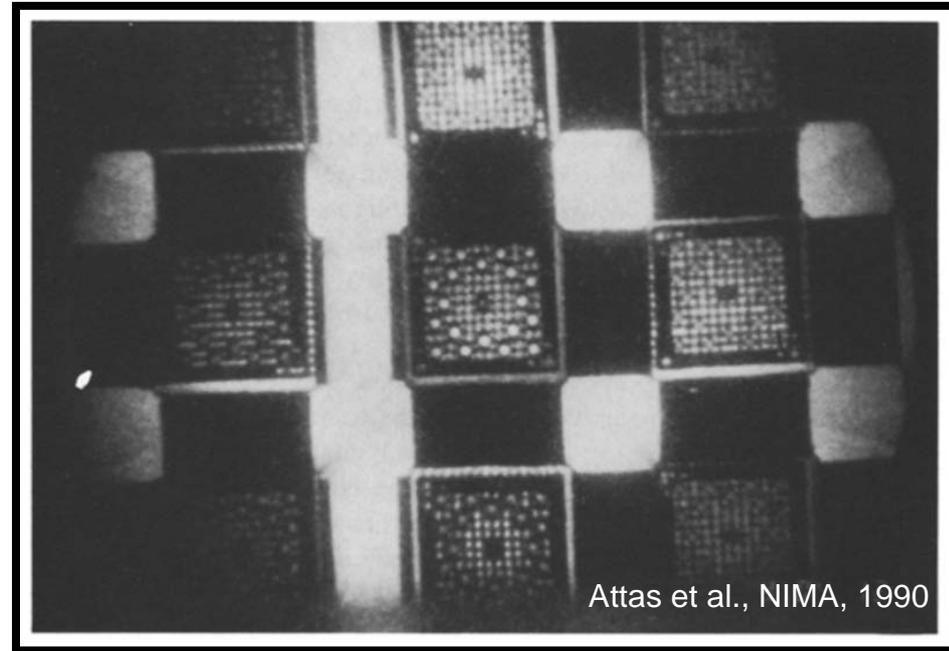
Cerenkov Viewing Device

- **Benefits:**

- Tested, validated method with reliable history of use
- Readily detects missing fuel rods
- Burnup and cooling time verification
- Indirect measurement method, meaning fuel assemblies may remain in storage positions

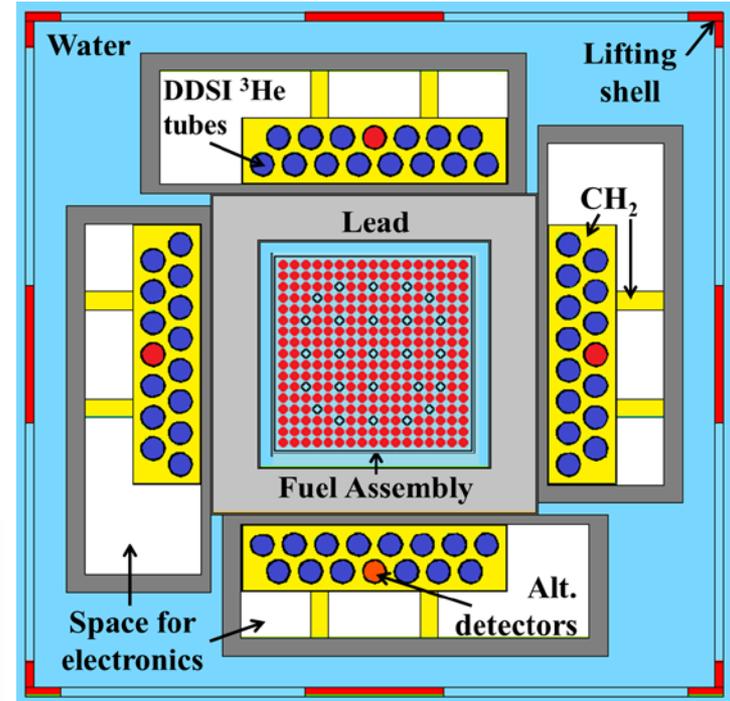
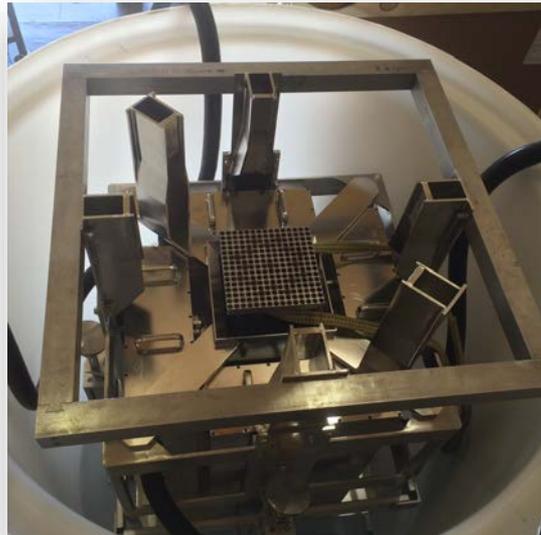
- **Drawbacks:**

- Murky water or weak Cerenkov signals can inhibit ability to use CVDs
- Neighboring assemblies in pool can confuse measurement
- Limited to certain burnups and cooling times due to required signal strength
- Potentially easy to fool with cutoff pins or fake fuel rods



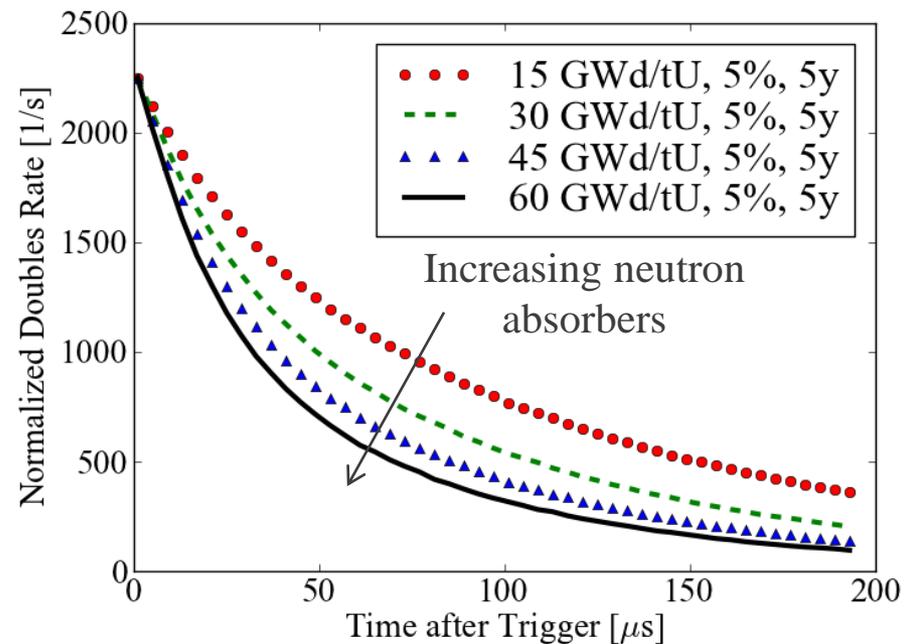
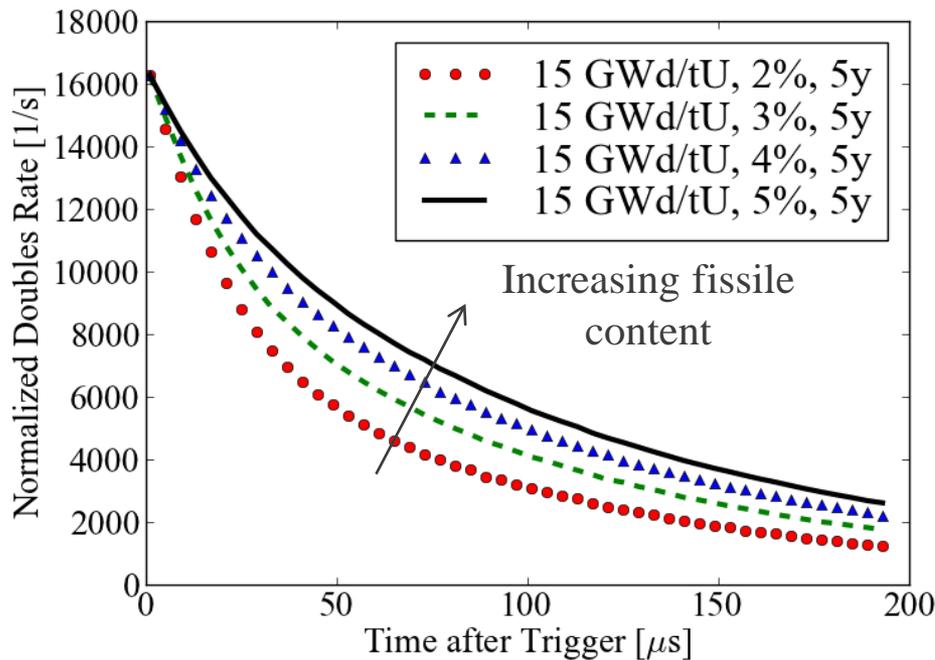
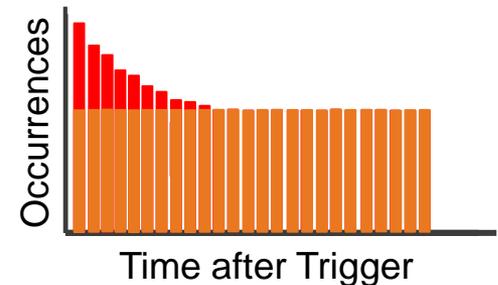
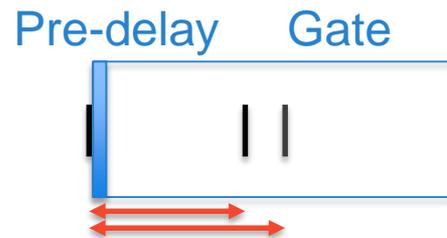
Differential Die-Away Self-Interrogation

- Spontaneous fission neutrons from ^{244}Cm , ^{240}Pu in spent fuel thermalize in water and interrogate fuel pins
- Neutron coincidence counting: aim to detect two neutrons that are temporally correlated
 - Same fission event, same fission chain
- Record times of neutron detections
 - *list-mode data*



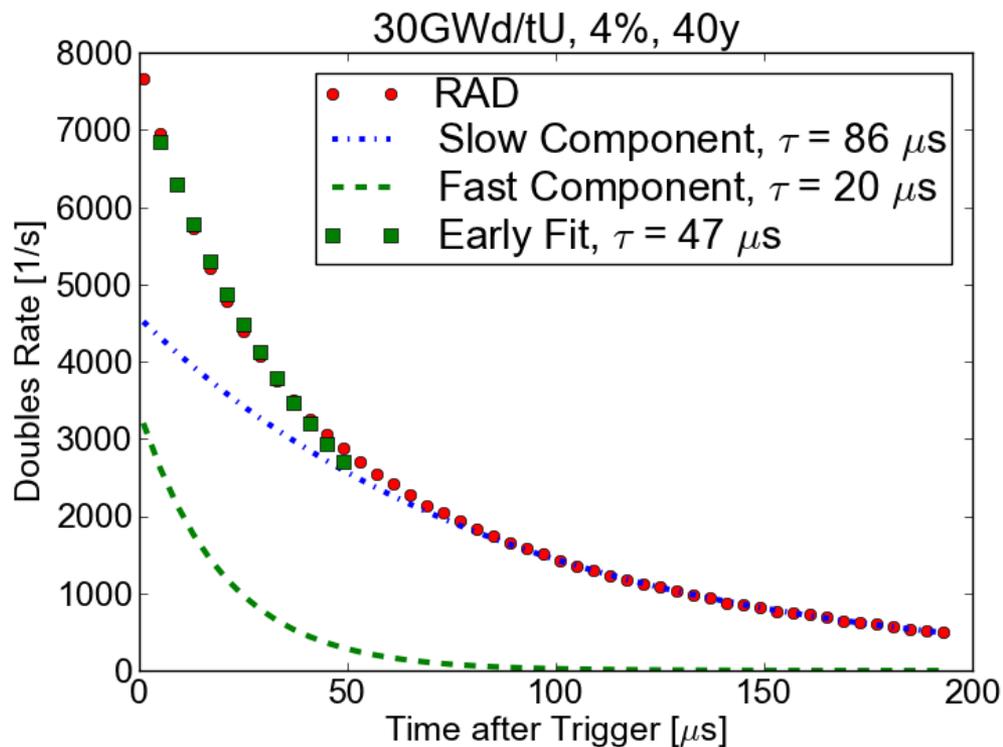
Differential Die-Away Self-Interrogation

- Rossi-alpha distribution is a histogram of the times between the trigger and each neutron in the gate



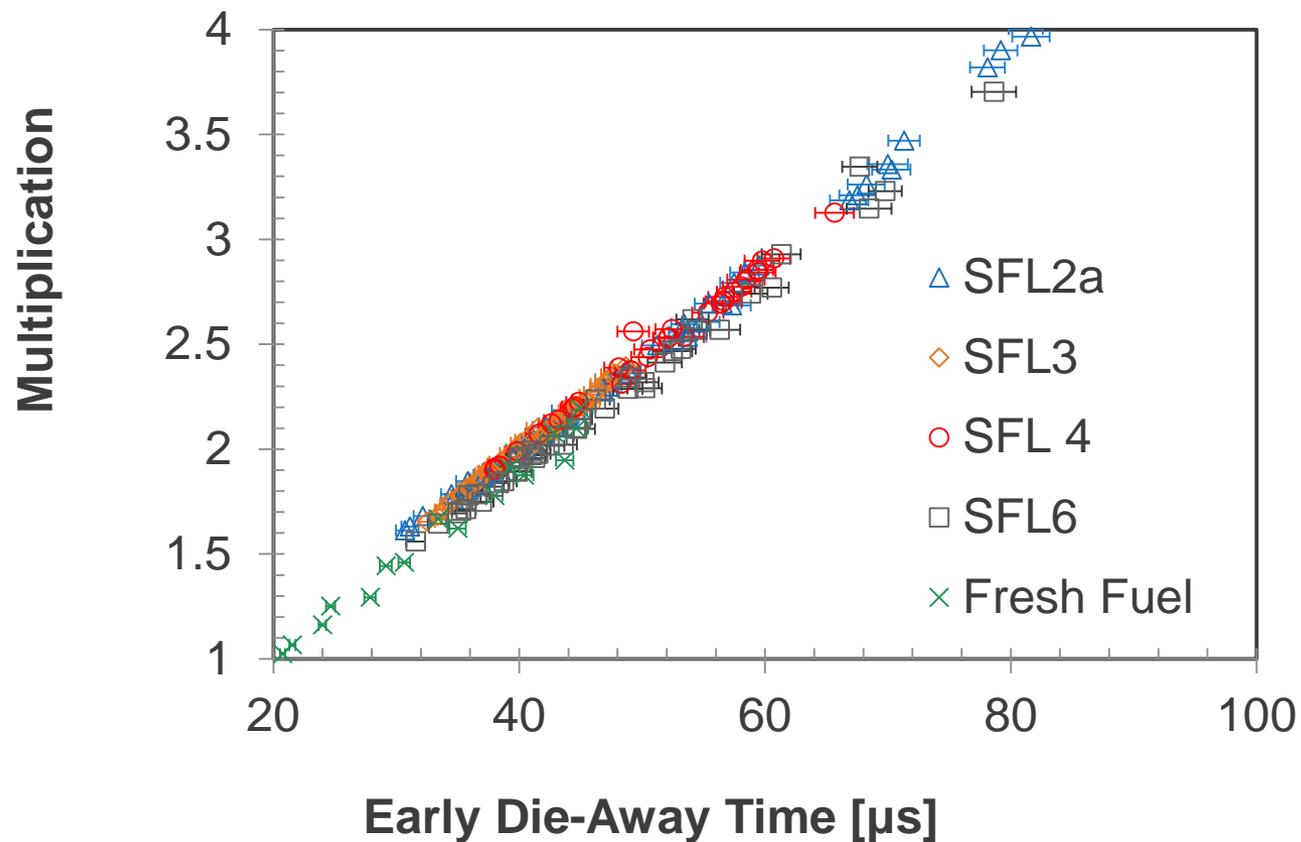
DDSI Analysis

- **RAD can be broken down into fast and slow components**
 - Fast component comes from detector die-away time
 - Slow component comes from neutron lifetime in the fuel
- **Early time domain of RAD can be fit with single exponential to yield early die-away time**
 - $y=Ae^{-t/\tau}$



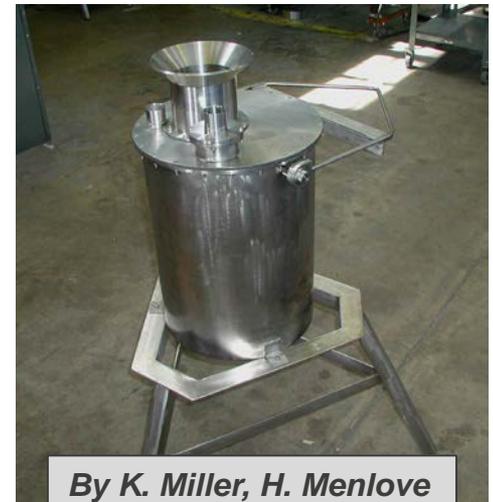
DDSI Analysis

- **Early die-away time is nearly linearly proportional to assembly multiplication**
 - Using this, one can determine whether pins have been removed



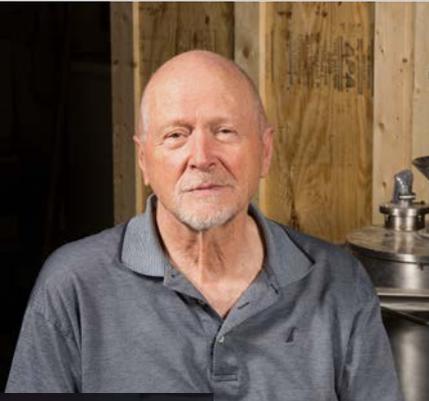
Advanced Experimental Fuel Counter

- **Designed for research reactors only**
- **System uses:**
 - Active and passive neutron coincidence counting;
 - An ion chamber for gross gamma-ray counting.
- **Measurement objective is to verify residual fissile mass (i.e., ^{235}U + ^{239}Pu) using active neutron interrogation**
- **Extended analysis of passive neutron and gamma-ray count rates helps verify declared burnup, cooling time, and initial enrichment**
- **Field trials have occurred as follows:**
 - 2006 High Flux Australian Reactor (HIFAR), Australia,
 - 2011 Institute of Nuclear Physics (INP), Uzbekistan, and
 - 2014 Institute of Nuclear Physics (INP), Uzbekistan.



By K. Miller, H. Menlove

Safeguards at LANL



**NUCLEAR
SAFEGUARDS**
LOS ALAMOS NATIONAL LABORATORY

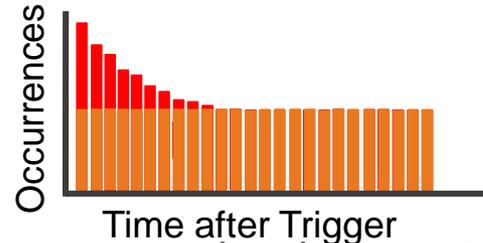
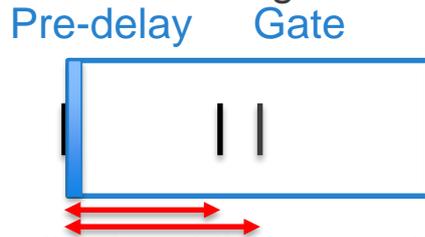


Thank you!

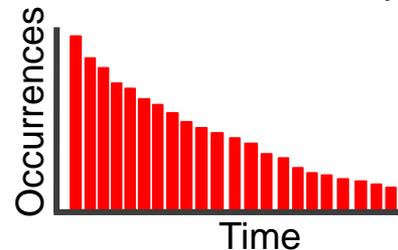
Neutron Counting: Distributions

- If you record the time of arrival of every neutron (list mode), several distributions can be produced:

– **Rossi-Alpha Distribution:** Histogram times between neutron arrivals within a certain gate



– **Time-Interval Distribution:** Histogram times between each subsequent neutron



– **Multiplicity Distribution:** Count number of neutrons in gate after trigger

